

8 Canada's RW

Canada's LLW Volume Projections to year 2025.

| Canadian nuclear industry | (m ³) | % |
|---|-------------------|------------|
| Refining | 65,000 | 18 |
| Fuel fabrication | 14,800 | 4 |
| Utilities | 156,500 | 42 |
| Isotopes and research | 61,200 | 16 |
| Licensed users | 12,900 | 3 |
| Industries using naturally radioactive feedstocks | 57,100 | 15 |
| Total | 367,500 | 100 |

Irradiated fuel bays at Ontario Hydro's NGS.

| Station | Type | Dimensions - m | | | C | ISD | BFD | LM |
|---------------|------|----------------|--------|-------|---------|---------|------|------|
| | | Width | Length | Depth | | | | |
| Pickering A/B | PIFB | 16.3 | 29.3 | 8.1 | 93/158 | 1972/83 | 1995 | E |
| | AIFB | 17 | 34 | 8.1 | 214 | 1978 | 1994 | E |
| Bruce A/B | PIFB | 10 | 41 | 6 | 21/36 | 1977/83 | 2002 | SS+E |
| | IFB | 18 | 46 | 9 | 352/330 | 1979/87 | 2002 | SS+E |
| Darlington | PIFB | 9.7 | 20.6 | 5 | 212 | 1987 | 1996 | SS |

C = capacity 1000's bundles, ISD = in-service date, BFD = bay fill date, LM = liner material (SS = Stainless steel, E = Epoxy)

Irradiated fuel bay purification system capacity.

| Station | Type | F-l/s | E | Type | F | E |
|---------------|------|-------|------|------|----|------|
| Pickering A/B | PIFB | 12/64 | IX | AIFB | 65 | F+IX |
| Bruce A/B | PIFB | 76/76 | IX | AIFB | 38 | IX |
| Darlington | PIFB | 92 | F+IX | | | |

F = flow rate, E = equipment (F = filters, IX = ion exchange)

Used-fuel centre life-cycle cost and labour requirements.

| Estimate | Cost - 1991 M\$ | | | Labour - person*years | | |
|-----------------------|-----------------|--------------|--------------|-----------------------|--------------|--------------|
| | low | nominal | high | low | nominal | high |
| Siting (23 a) | 1850 | 2180 | 3050 | 6880 | 8100 | 11330 |
| Construction (7 a) | 1540 | 1810 | 2530 | 6240 | 7340 | 10280 |
| Operation (41 a) | 6850 | 8060 | 11280 | 33880 | 39850 | 55800 |
| Decommissioning(16 a) | 1060 | 1250 | 1750 | 5720 | 6730 | 9430 |
| Closure (2 a) | 30 | 30 | 40 | 120 | 150 | 200 |
| Total | 11320 | 13320 | 18650 | 52840 | 62170 | 87040 |

Scaled nominal cost (M\$ 1991) and Duration (D in years) estimates for disposal vault capacities of 5, 7.5 and 10.1 Million used-fuel bundles at depth of 1000 m.

| Million of bundles | 5 | | 7.5 | | 10.1 | |
|--------------------|-----------|-------------|-----------|--------------|-----------|--------------|
| | D | Cost | D | Cost | D | Cost |
| Siting | 23 | 2140 | 23 | 2160 | 23 | 2180 |
| Construction | 5 | 1520 | 6 | 1630 | 7 | 1810 |
| Operation | 20 | 4060 | 30 | 6040 | 41 | 8060 |
| Decommissioning | 13 | 940 | 15 | 1090 | 16 | 1250 |
| Closure | 2 | 30 | 2 | 30 | 2 | 30 |
| Total | 63 | 8680 | 76 | 10950 | 89 | 13320 |

Comparison of nominal cost (M\$ 1991) and schedule durations (D in years) for a disposal centre with a vault at depths of 500 and 1000 m (Capacity = 10.1 million used-fuel bundles).

| Depth = | 500 m | | 1000 m | |
|-----------------|-------|-------|--------|-------|
| | D | Cost | D | Cost |
| Siting | 22 | 2110 | 23 | 2180 |
| Construction | 7 | 1780 | 7 | 1810 |
| Operation | 41 | 8060 | 41 | 8060 |
| Decommissioning | 14 | 1130 | 16 | 1250 |
| Closure | 2 | 30 | 2 | 30 |
| Total | 86 | 13110 | 89 | 13320 |

Percentage of contaminants present in different compartments at 1E+4 a.

| Amount remaining in | I-129 | C-14 | Tc-99 | U-238 |
|---------------------|-------|------|-------|-------|
| Containers | 96.06 | 28.0 | 91.0 | 99.99 |
| Backfill + Buffer | 3.85 | 1.9 | 5.79 | 2E-7 |
| Geosphere | 0.07 | 0.02 | 0 | 0 |
| Released biosphere | 0.02 | 0 | 0 | 0 |

Mean concentrations (MC) of contaminants in soil and water and their environmental increments (EI).

| | 129-I | | 14-C | |
|------------|-------|------|------|------|
| Medium | MC | EI | MC | EI |
| Soil Bq/kg | 2 | 1E-5 | 9E-3 | 9E-3 |
| Water Bq/L | 3E-3 | 4E-8 | 5E-4 | 2E-5 |

Arithmetic mean of the maximum doses to four hypothetical organisms estimated in 1000 simulations for a 100 000-year simulation time (mGy/a).

| Nuclide | Plant | Fish | Mammal | Bird |
|---------|-------|------|--------|------|
| 129-I | 4E-3 | 3E-3 | 1E-2 | 5E-2 |
| 14-C | 2E-4 | 2E-2 | 5E-4 | 5E-4 |
| Total | 4E-3 | 2E-2 | 1E-2 | 5E-2 |

Percentage of a nuclide released by a barrier over 100 000 years.

| Nuclide | T - a | Fuel | Container | Vault | Rock |
|---------|---------|---------|-----------|---------|---------|
| 3-H | 12.4 | 30 | <<0.001 | <<0.001 | <<0.001 |
| 90-Sr | 29.1 | 0.05 | <<0.001 | 1 | <<0.001 |
| 39-Ar | 269 | 8 | 0.08 | <<0.001 | <<0.001 |
| 14-C | 5730 | 6 | 60 | 0.8 | 0.007 |
| 239-Pu | 2.41E+4 | <<0.001 | 100 | <<0.001 | <<0.001 |
| 99-Tc | 2.13E+5 | 6 | 100 | <<0.001 | 0.1 |
| 129-I | 1.57E+7 | 6 | 100 | 10 | 5 |
| Br | stable | 6 | 100 | 10 | 5 |
| Sb | stable | <<0.001 | 100 | 0.003 | 5 |

Maximum Estimated Risk (MER) and Time of Occurrence (TO) from four human intrusion scenarios.

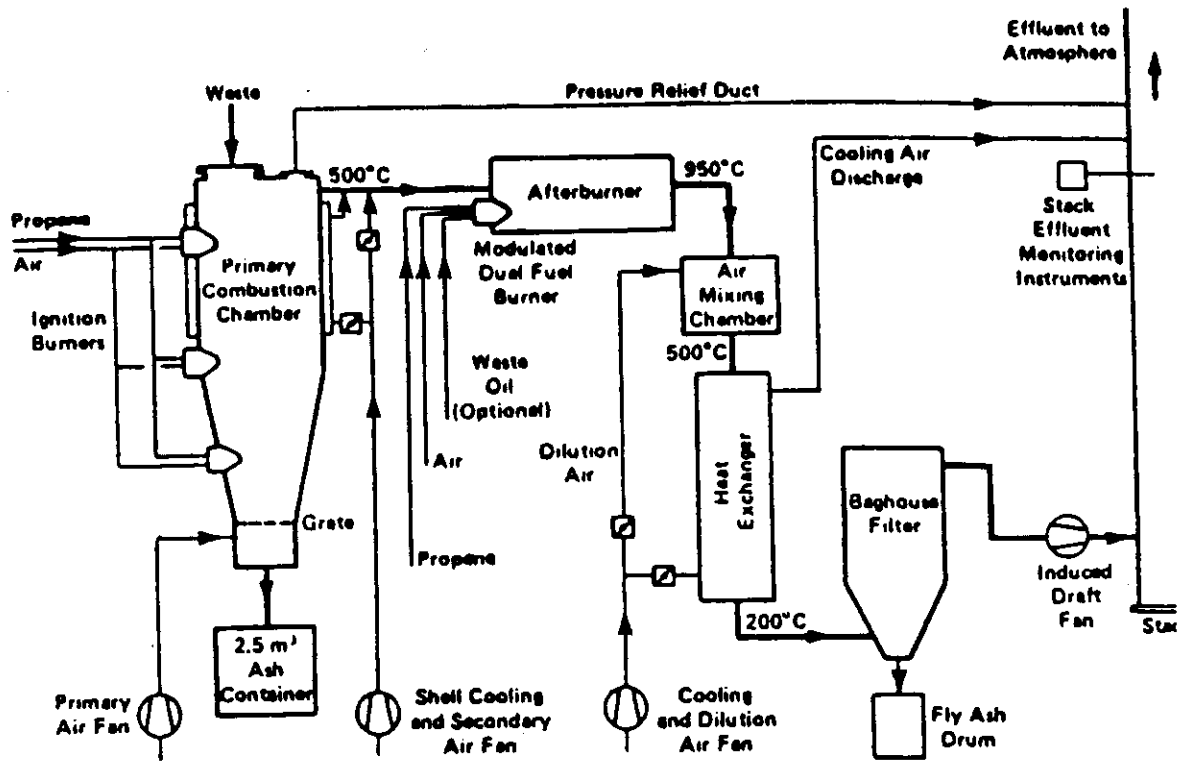
| Scenario | MER/y | TO - y |
|------------------|-------|--------|
| Drilling | 3E-10 | 40 |
| Core Examination | 9E-11 | 500 |
| Construction | 4E-13 | 3000 |
| Resident | 3E-10 | 150 |

Amounts of contaminants (in mol) present in different compartments at 1E+5 a.

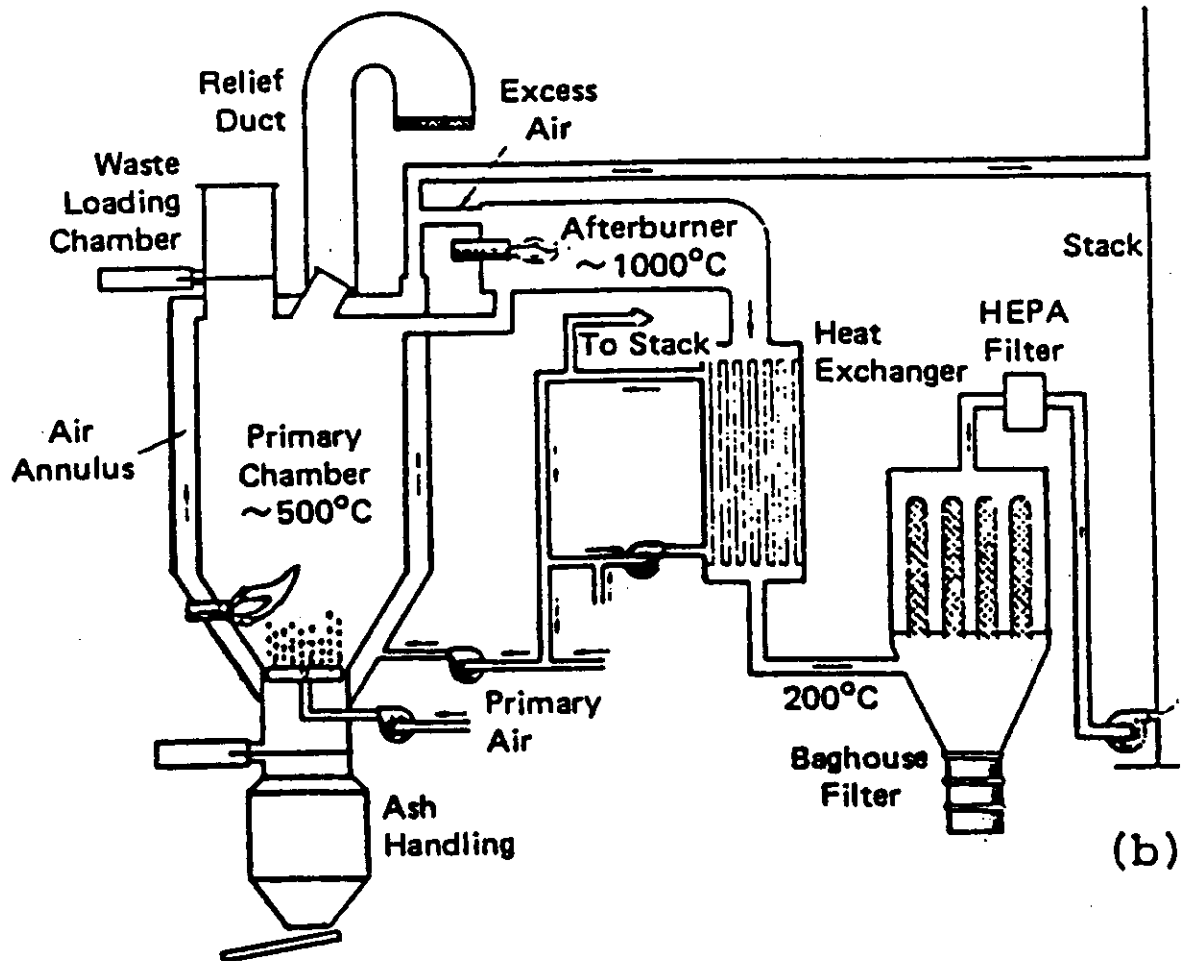
| Amount in | Br | C-14 | I-129 | Kr-81 | Pu-239 | U-238 |
|------------|-------|-------|-------|--------|--------|-------|
| Inventory* | 11000 | 3000 | 56000 | 0.011 | 19E+5 | 67E+7 |
| Containers | 9900 | 0.015 | 52000 | 0.0072 | 1E+5 | 67E+7 |
| Buffer | 0 | 0 | 0 | 7E-5 | 5E-4 | 4.2 |
| Backfill | 590 | 16E-4 | 3100 | 5E-4 | 2E-4 | 8E-3 |
| Vault | 11000 | 16E-3 | 55000 | 8E-3 | 1E+5 | 67E+7 |
| Released** | 2E-2 | 81E-8 | 0.28 | 1E-8 | 0 | 0 |

* initial, ** to biosphere.

LI



(a)



(b)

Figure 1: LLW Incinerators in Canada (a) Ontario Hydro (b) CRNL

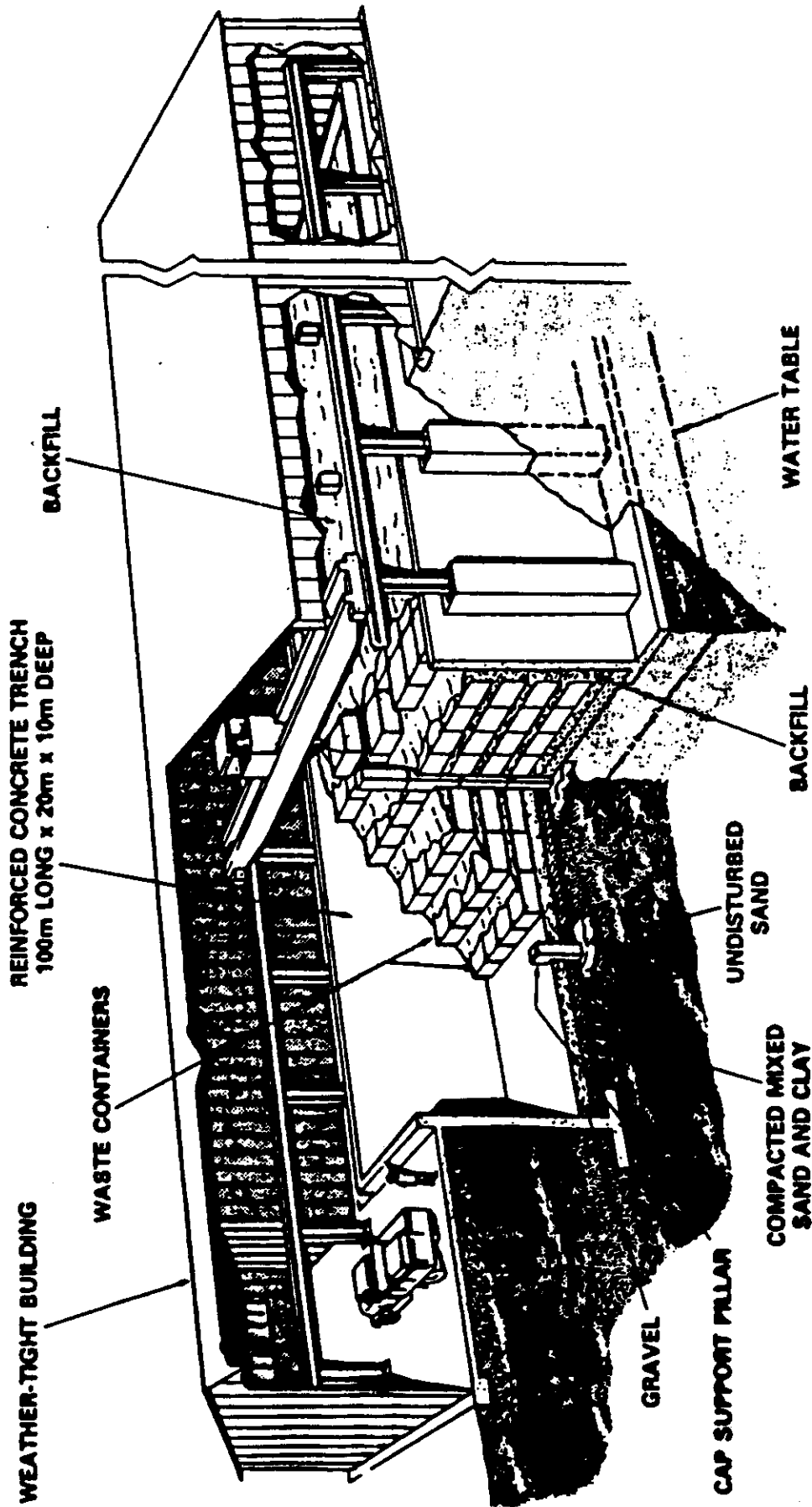
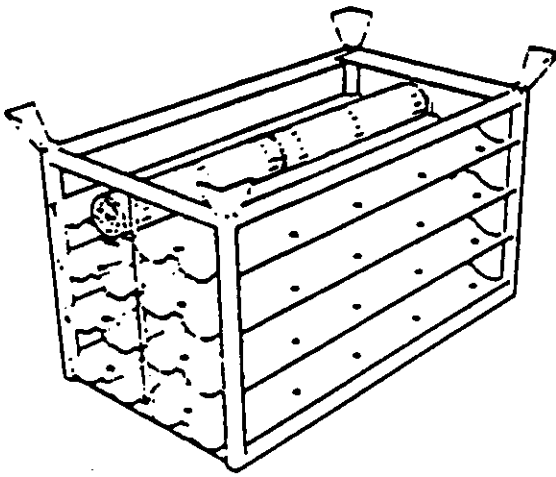


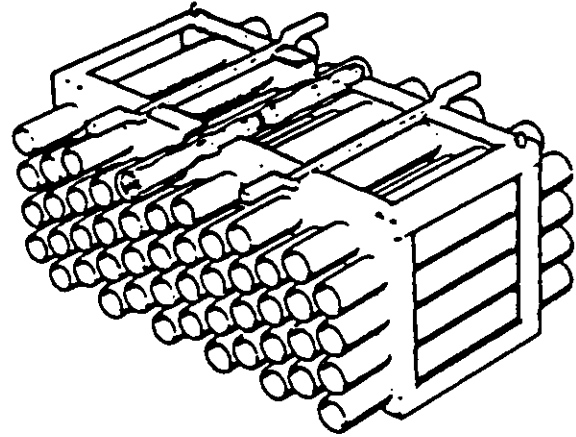
Figure 2: CRNL shallow land burial (slb) facility.

L-B

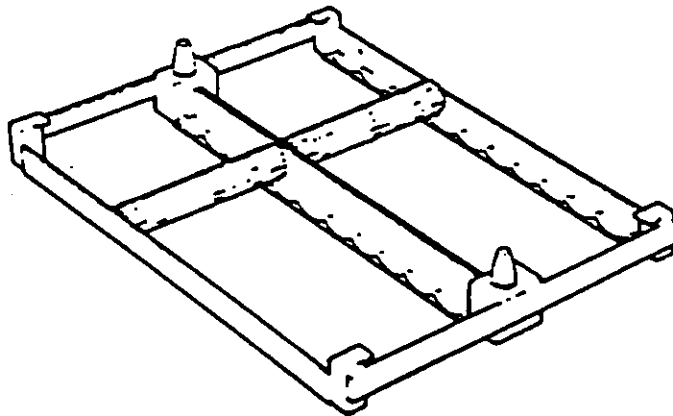
FS



**Pickering GS
Fuel Storage Basket**
(32 Bundle Capacity, Storage
Density = 1393 kg U/m³)



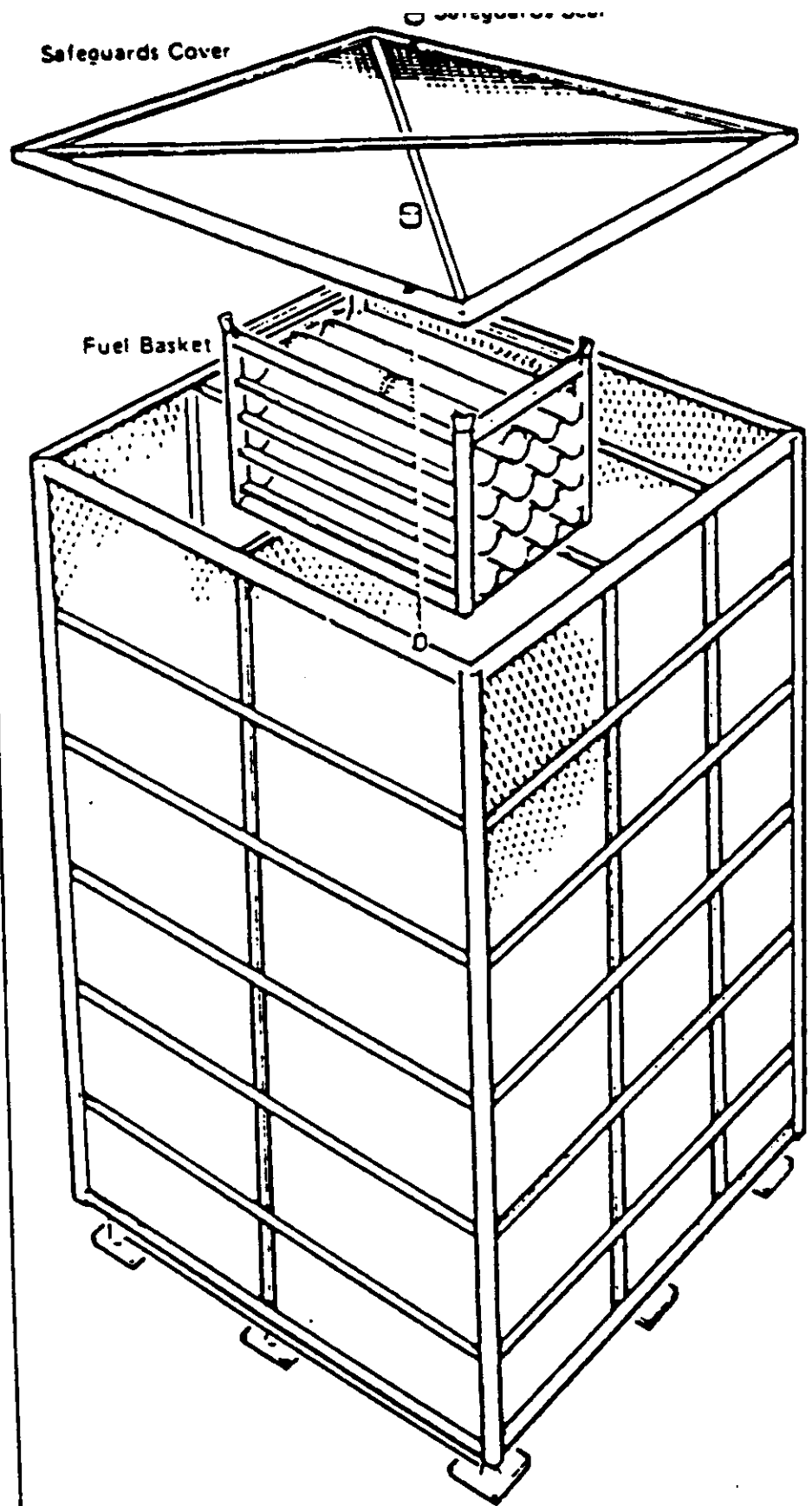
Shipping and Storage Module
(96 Bundle Capacity, Storage
Density = 2189 kg U/m³)



**Bruce GS
Irradiated Fuel Storage Tray**
(20 Bundle Capacity, Storage
Density = 1683 kg U/m³)

Figure 2 Ontario Hydro Irradiated Fuel Storage Containers

FSAI



Safeguards Cover

Fuel Basket

Figure 3 Pickering NGS-A Irradiated Fuel Stacking Frame

FSAI

F1

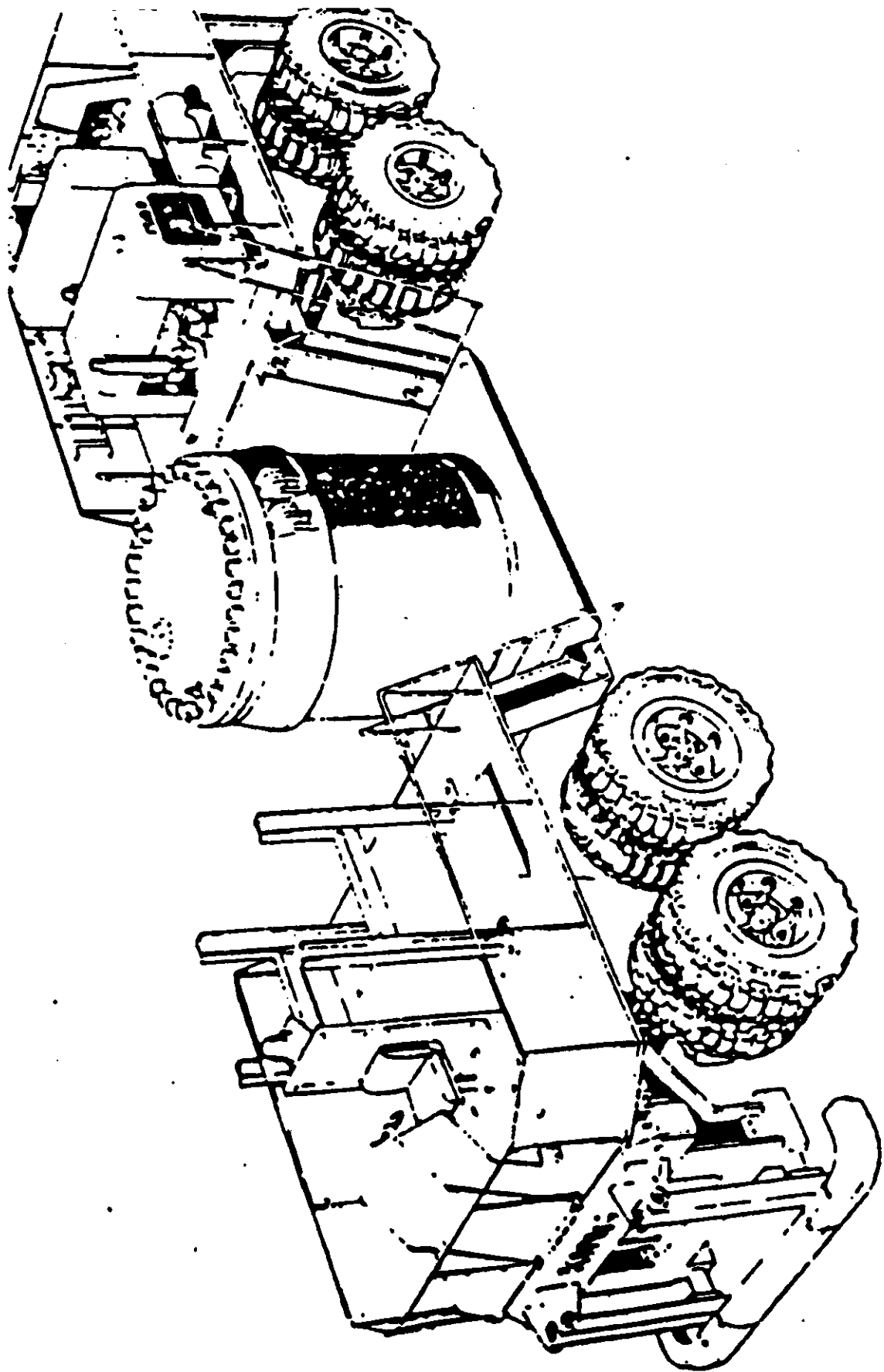


FIGURE 1 - CHASSIS - 1955

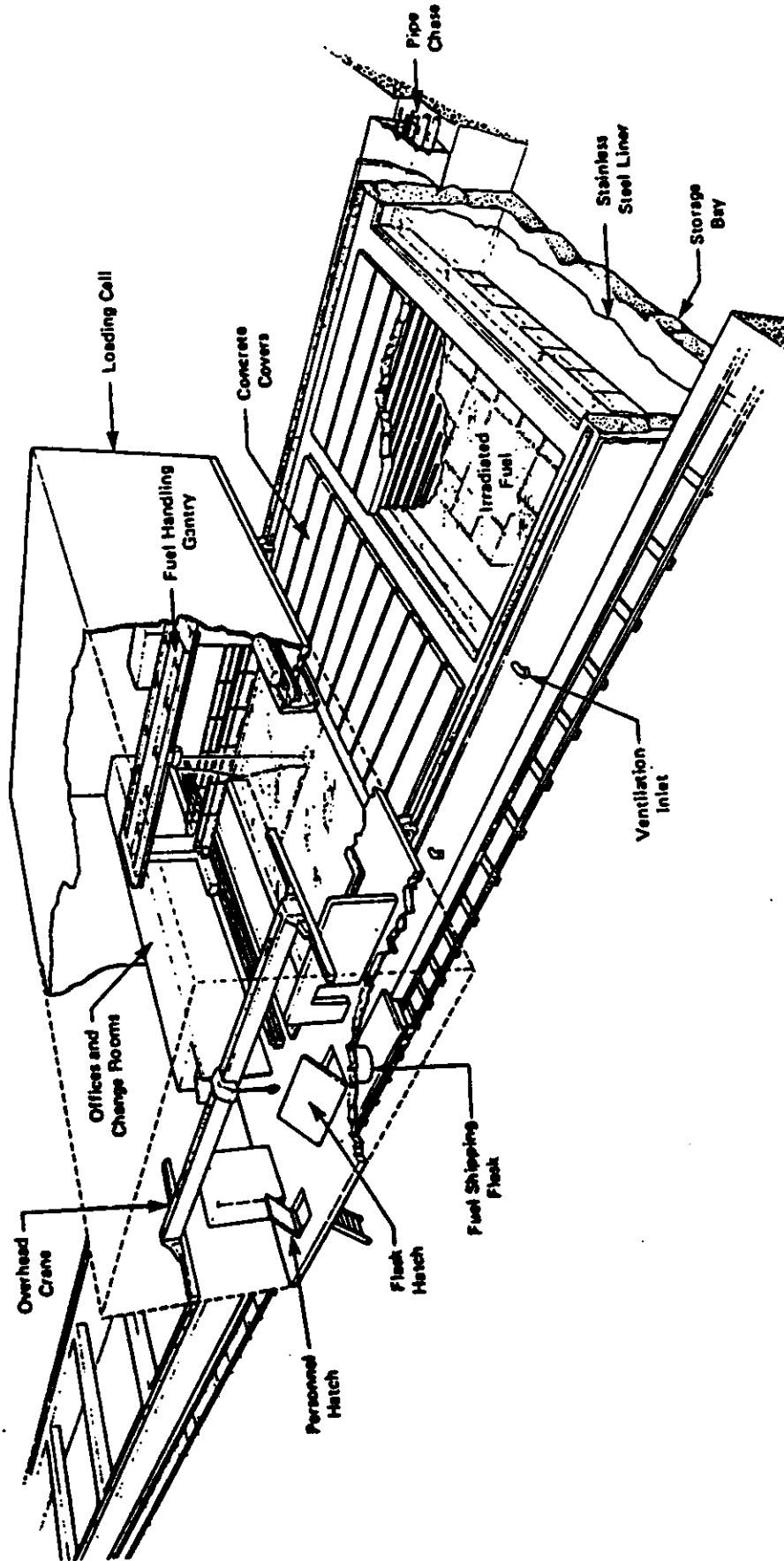


FIGURE 3: CENTRAL POOLS - CUTAWAY (From Ref. 5). AELL-6191

FSH

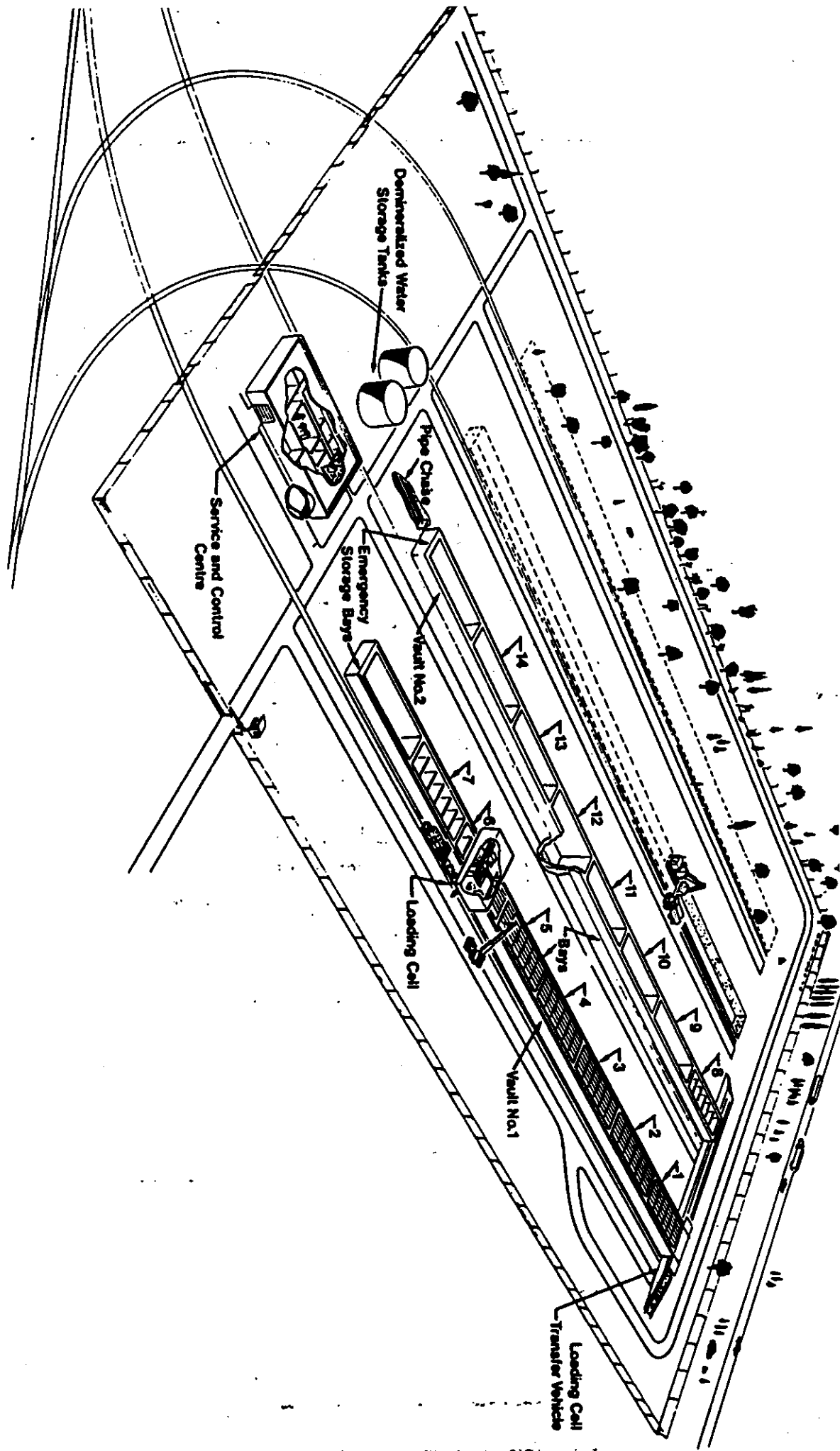


FIGURE 4: SPENT FUEL STORAGE FACILITY.

AEEL6191

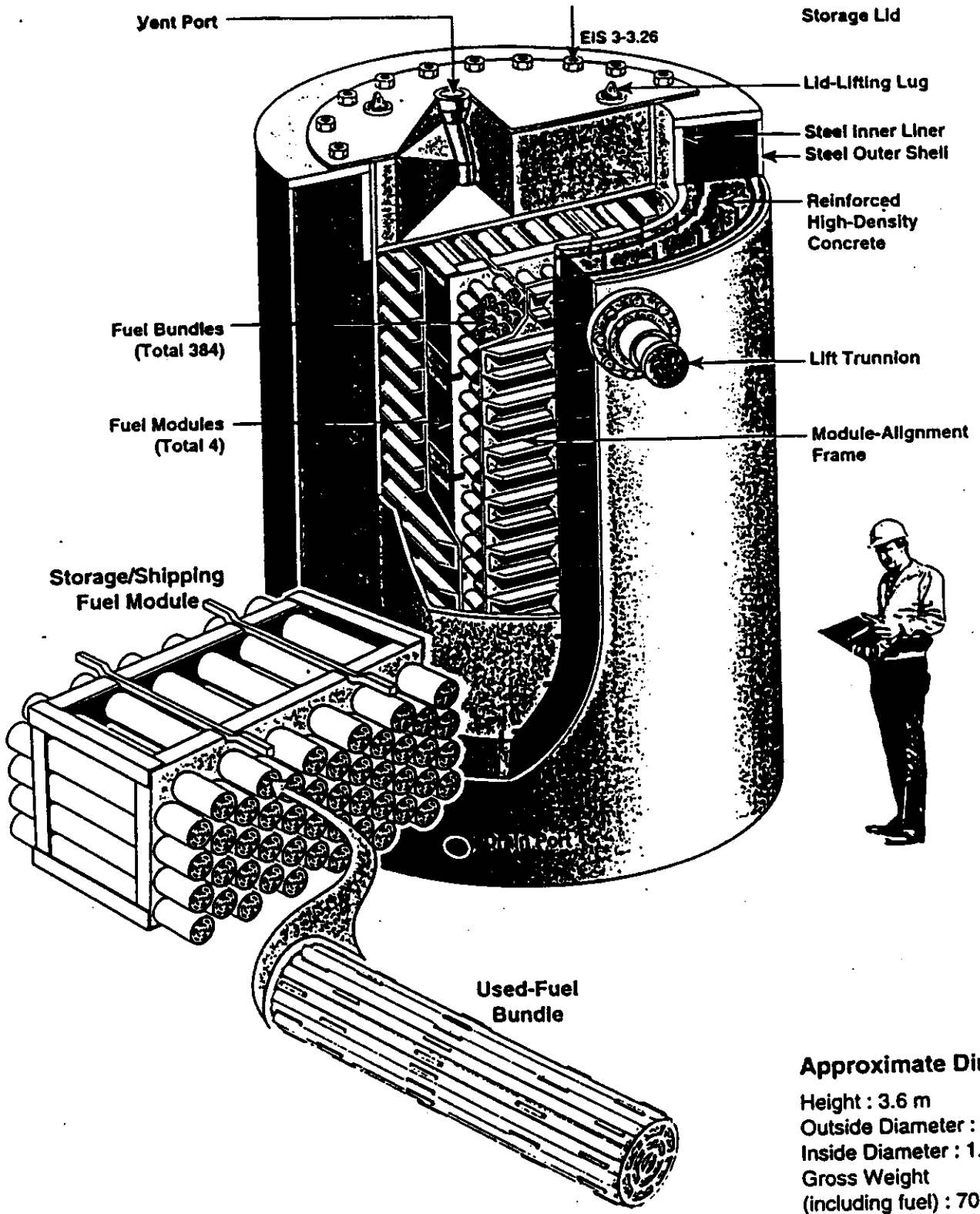


FIGURE 3-26: Ontario Hydro Dry Storage Container, Original Cylindrical Design

CSA

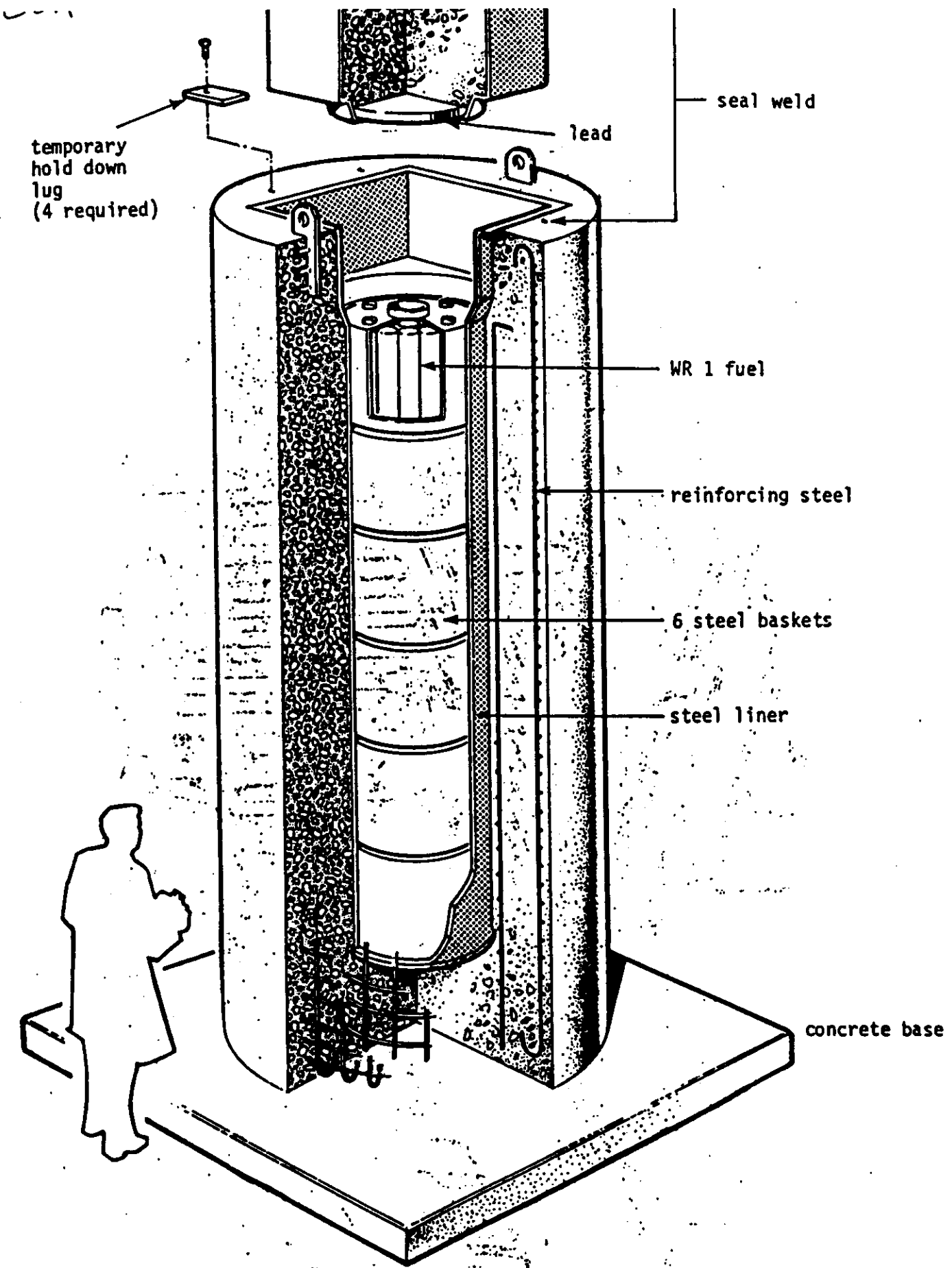


FIGURE 7: CYLINDRICAL CONCRETE CANISTER

AECL 6191

C3A

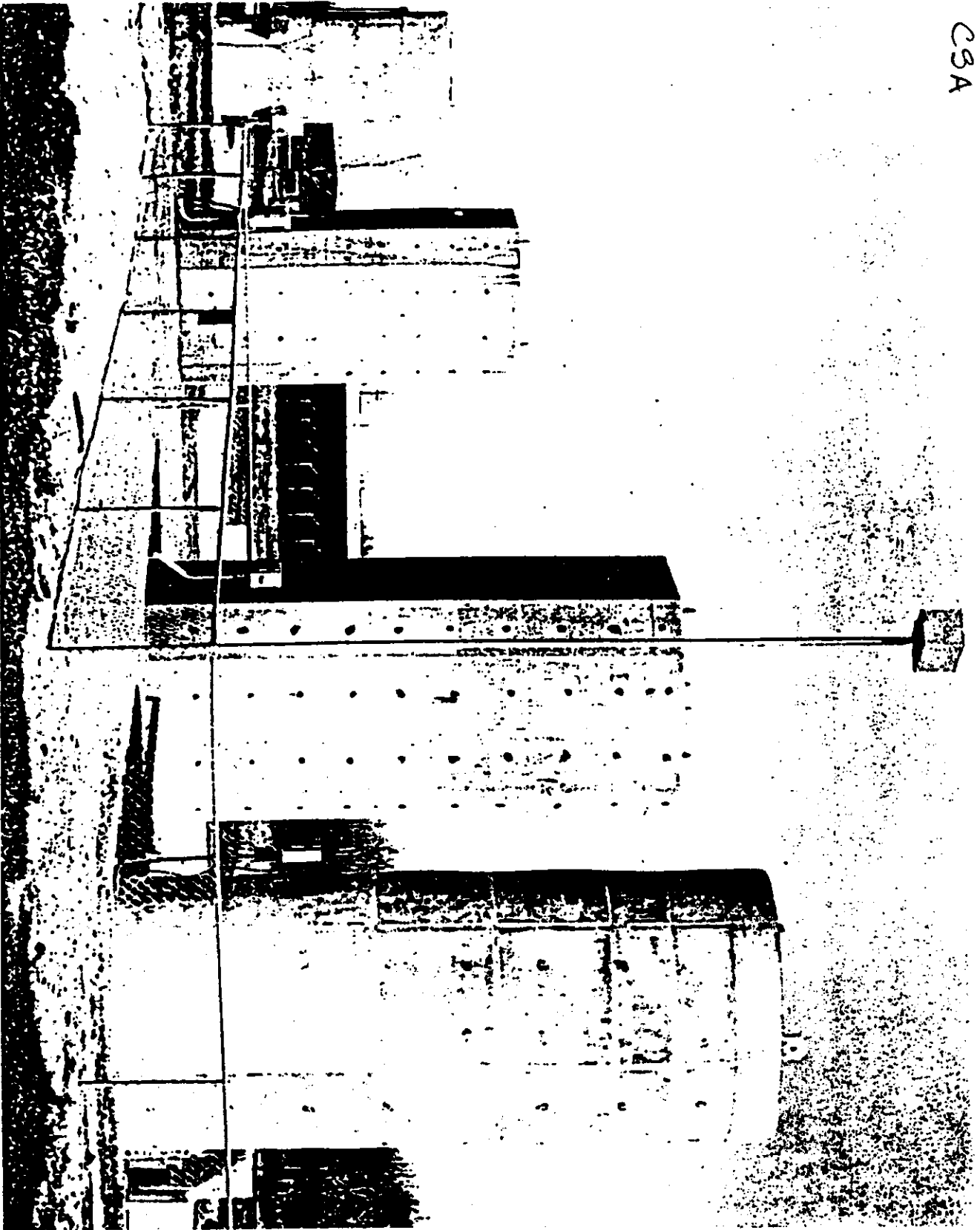


FIGURE 10: CANISTERS IN OPERATION.

NEEL6191

C3

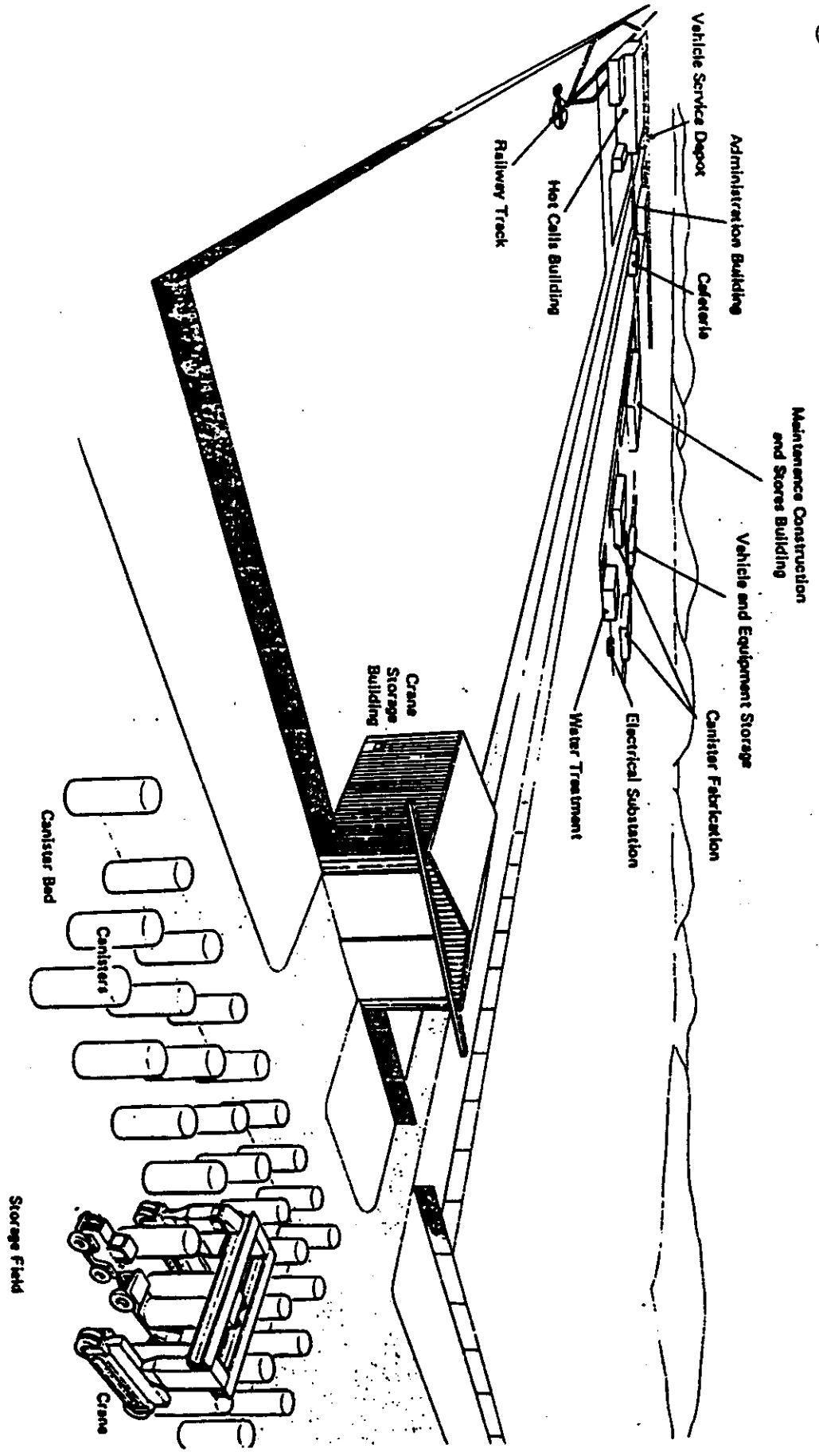
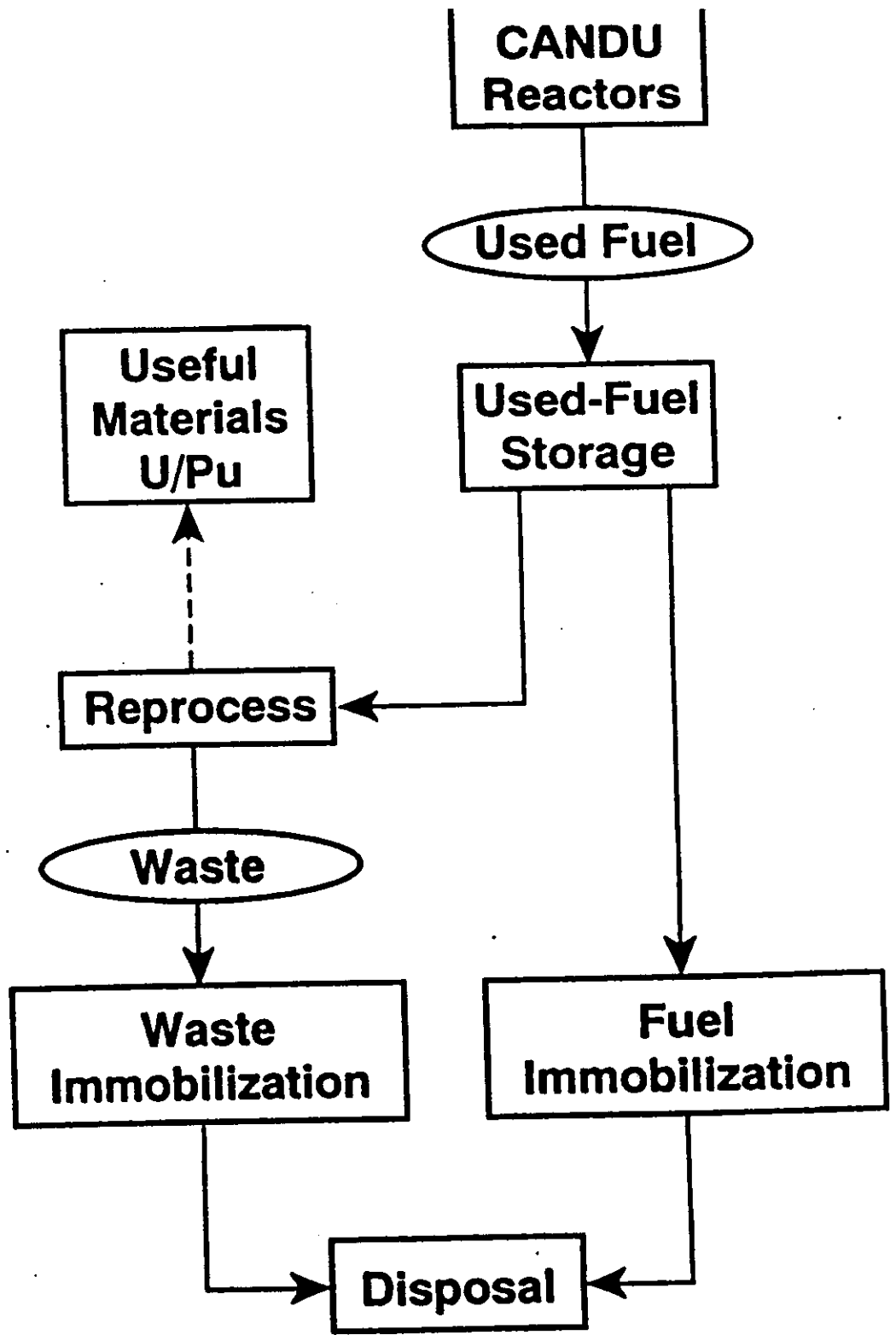


FIGURE 8: CANISTER STORAGE FACILITY.

AEEL 61-11

F 3-2.1



EIS 3-2.1

F 3-2.1
Options for the Storage and Disposal of

HDMMO

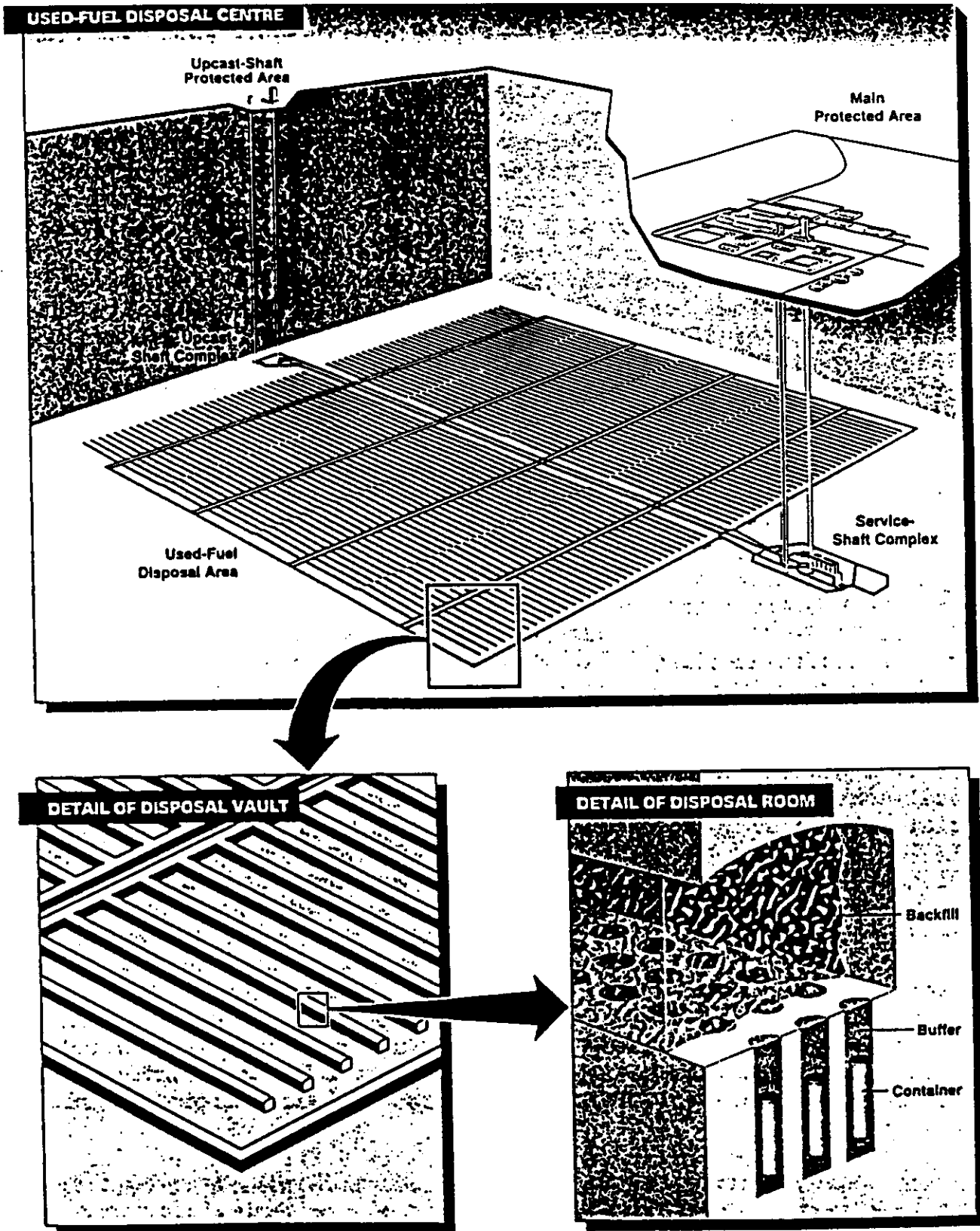
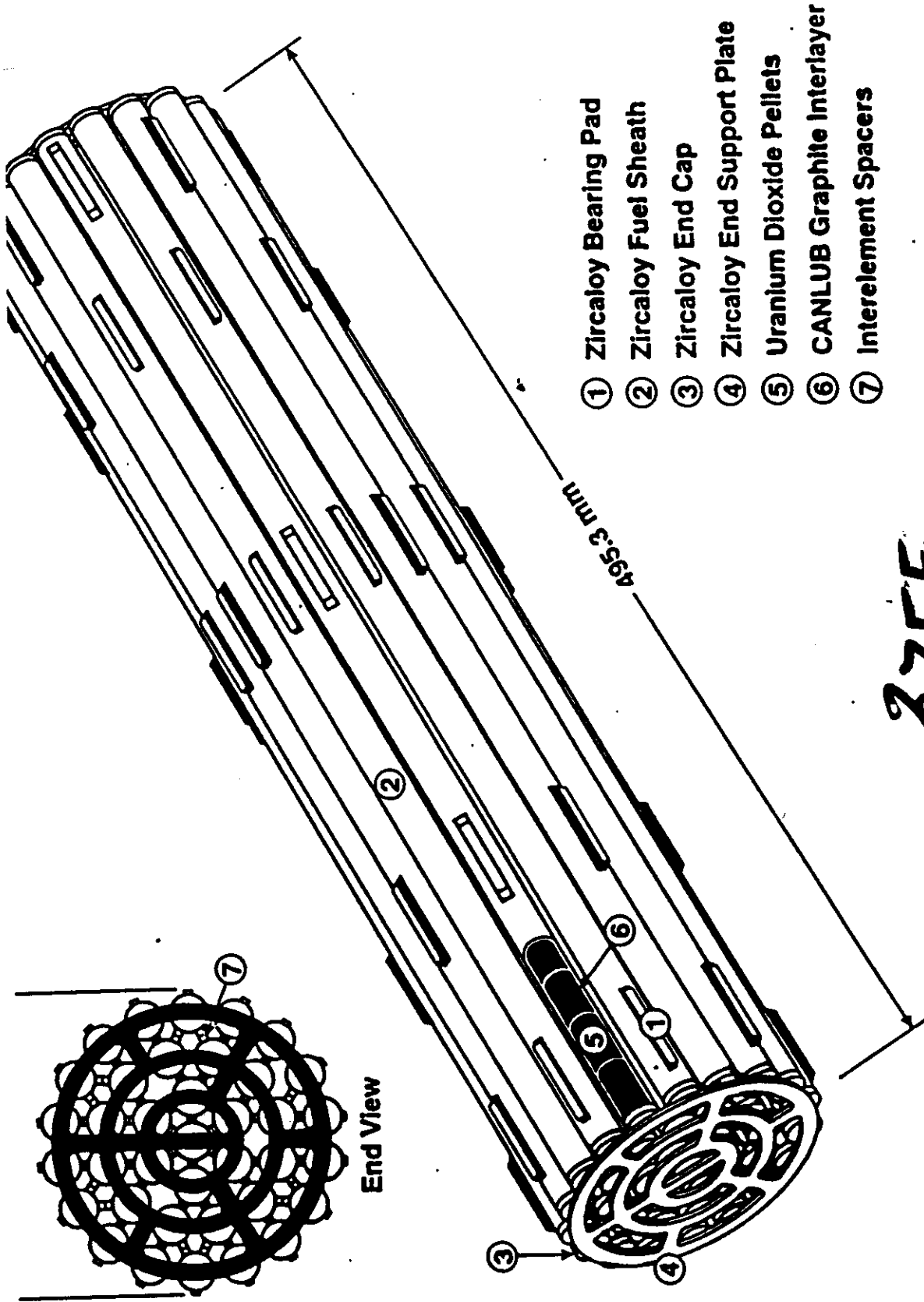


FIGURE ES-4: Used-Fuel Disposal Centre Perspective



- ① Zircaloy Bearing Pad
- ② Zircaloy Fuel Sheath
- ③ Zircaloy End Cap
- ④ Zircaloy End Support Plate
- ⑤ Uranium Dioxide Pellets
- ⑥ CANLUB Graphite Interlayer
- ⑦ Interelement Spacers

37FE

FIGURE ES-1: Typical CANDU Fuel Bundle for Bruce Nuclear Generating Station
 (after AECL CANDU et al. 1992)

of 685 GJ/kg U and cooled for 10 a after their discharge from a nuclear

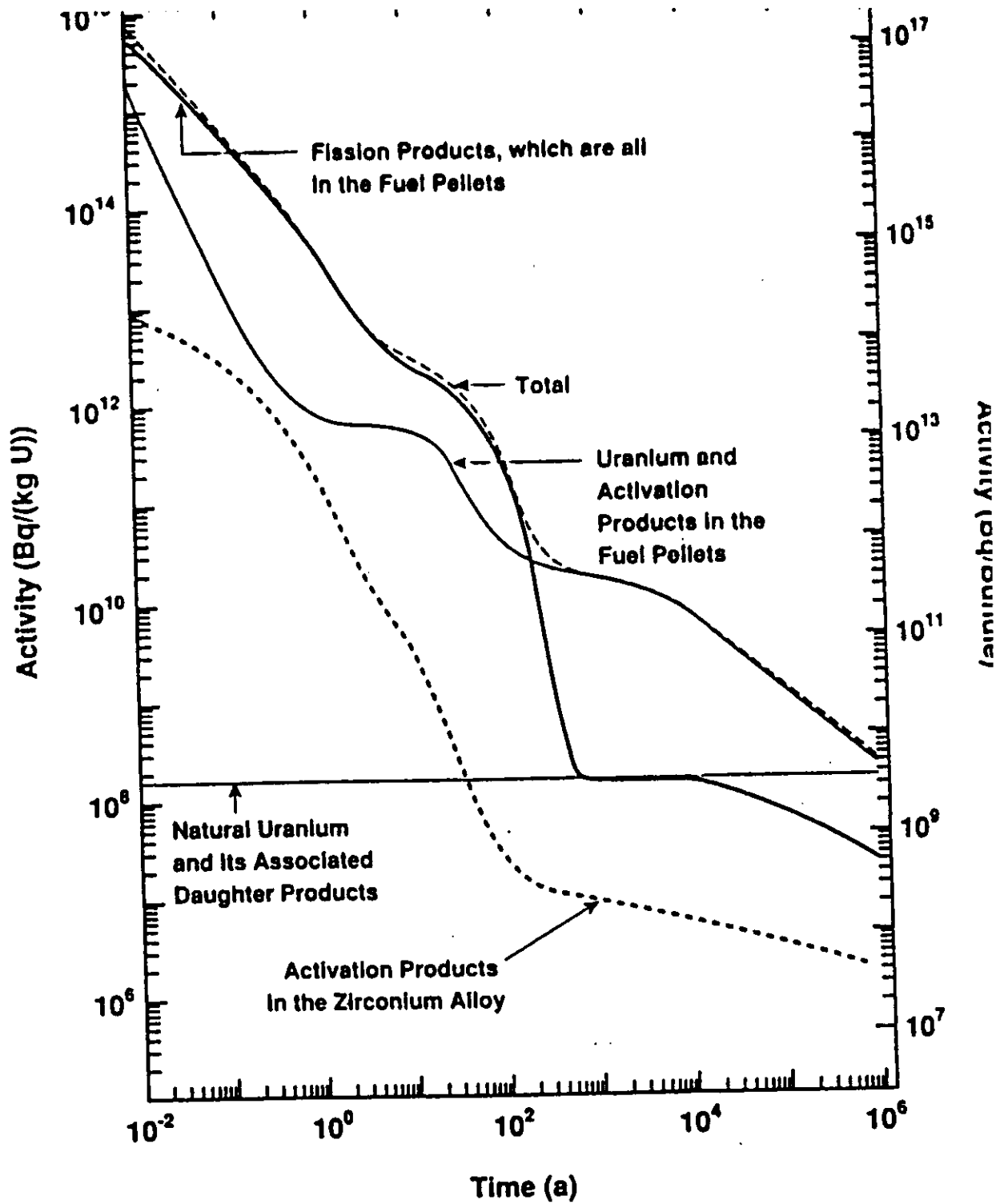


FIGURE 2-5: ACTIVITY OF THE USED FUEL SPECIFIED FOR THE CASE STUDIES (LOGARITHMIC SCALES)

FDH

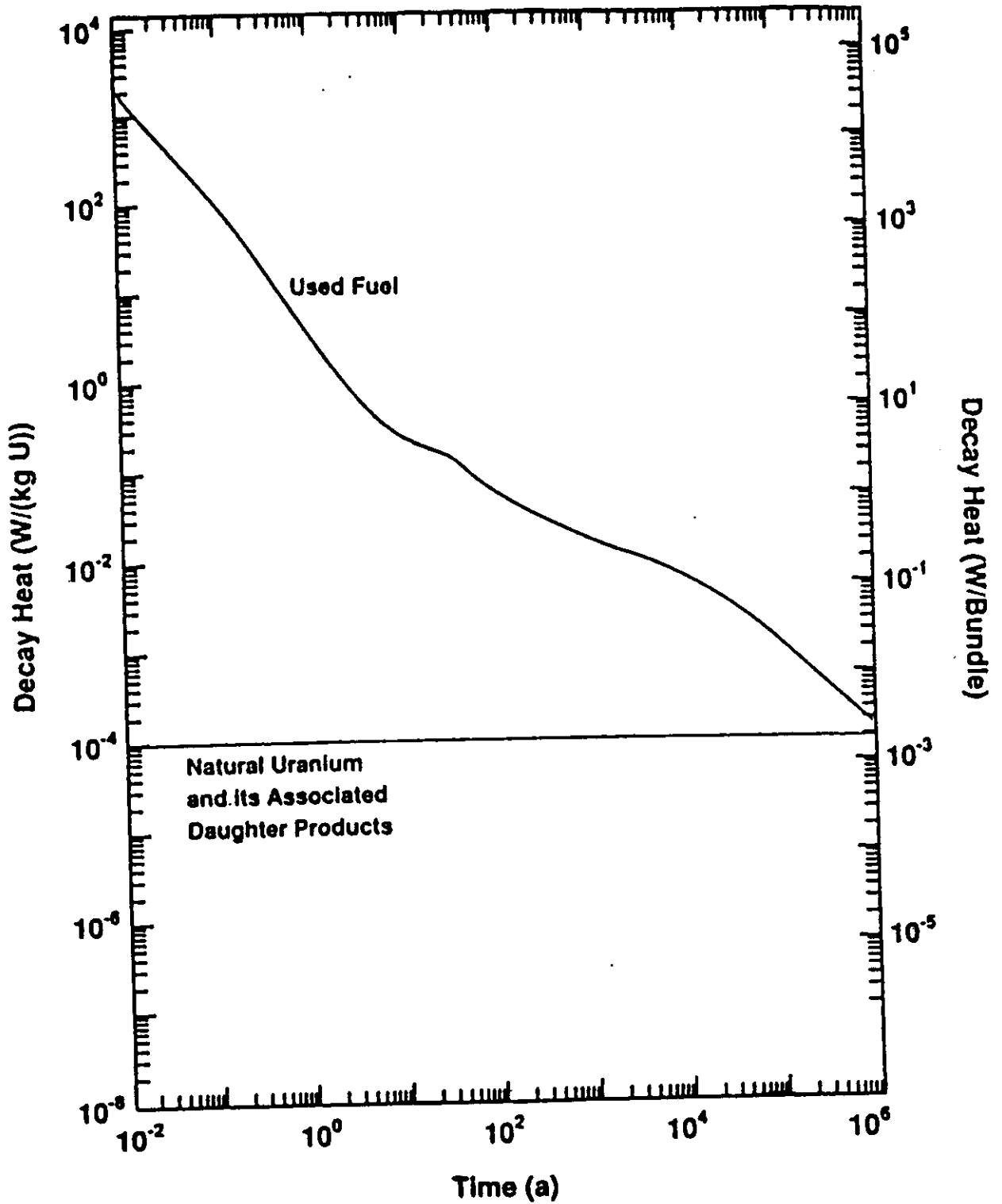


FIGURE 2-7: HEAT FROM THE USED FUEL SPECIFIED FOR THE CASE STUDIES (LOGARITHMIC SCALES)

FDH

RD&T

EIS 2.10

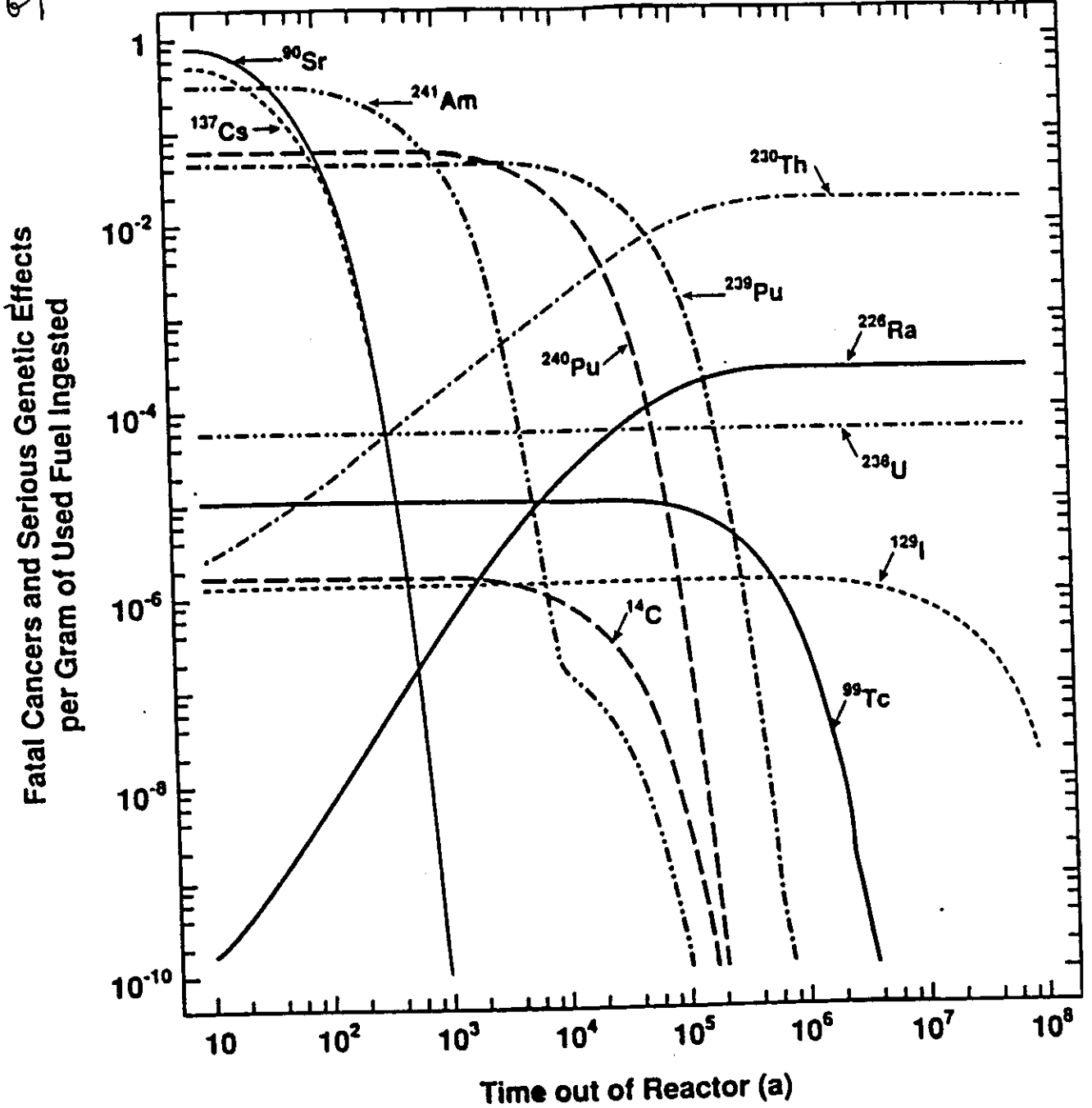
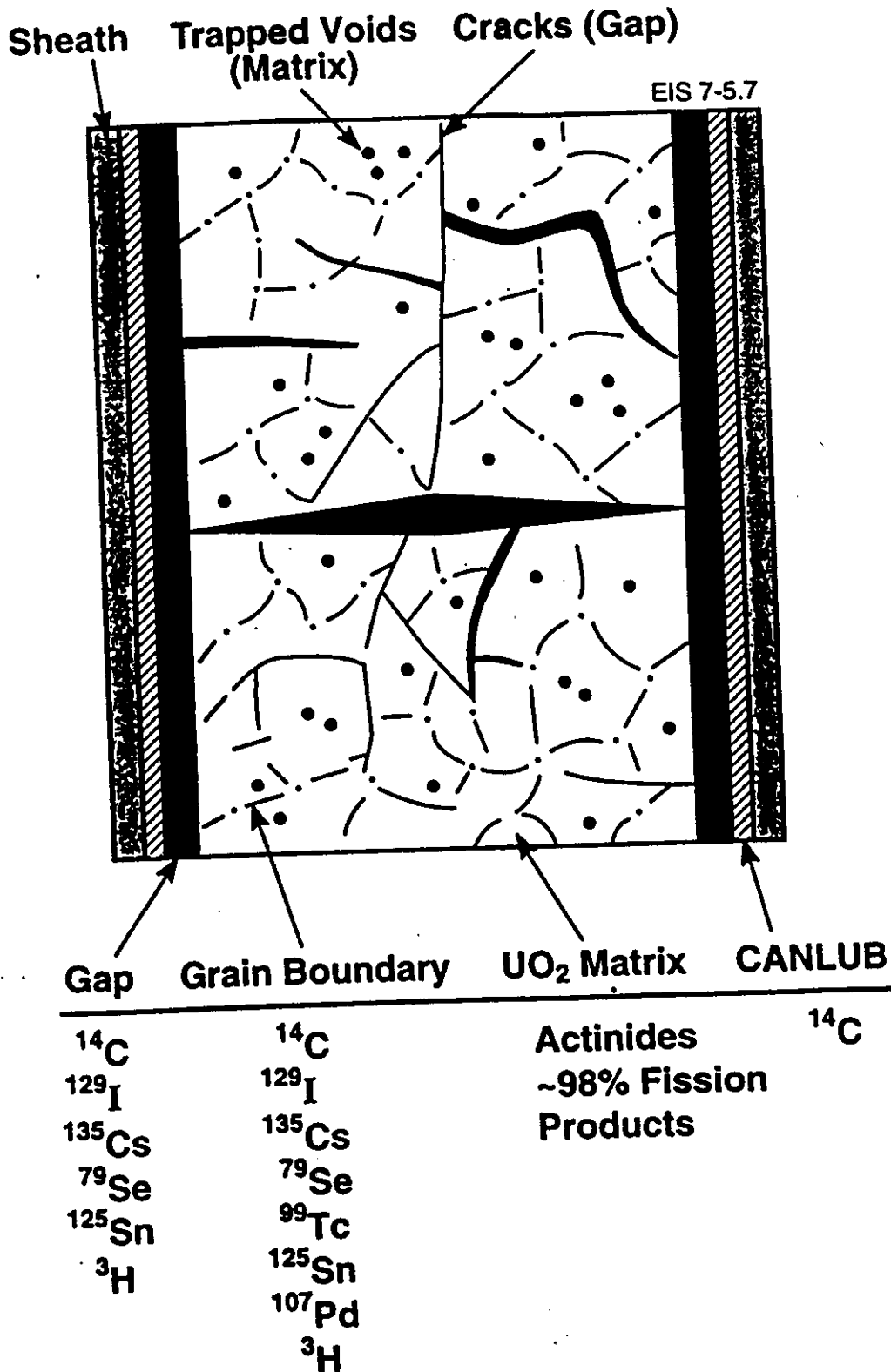


FIGURE 2-10: RADIOTOXICITY OF VARIOUS RADIONUCLIDES IN USED CANDU F



Conceptual Distribution of Some Fission and Activation Products Within a Used-Fuel Element

3DB

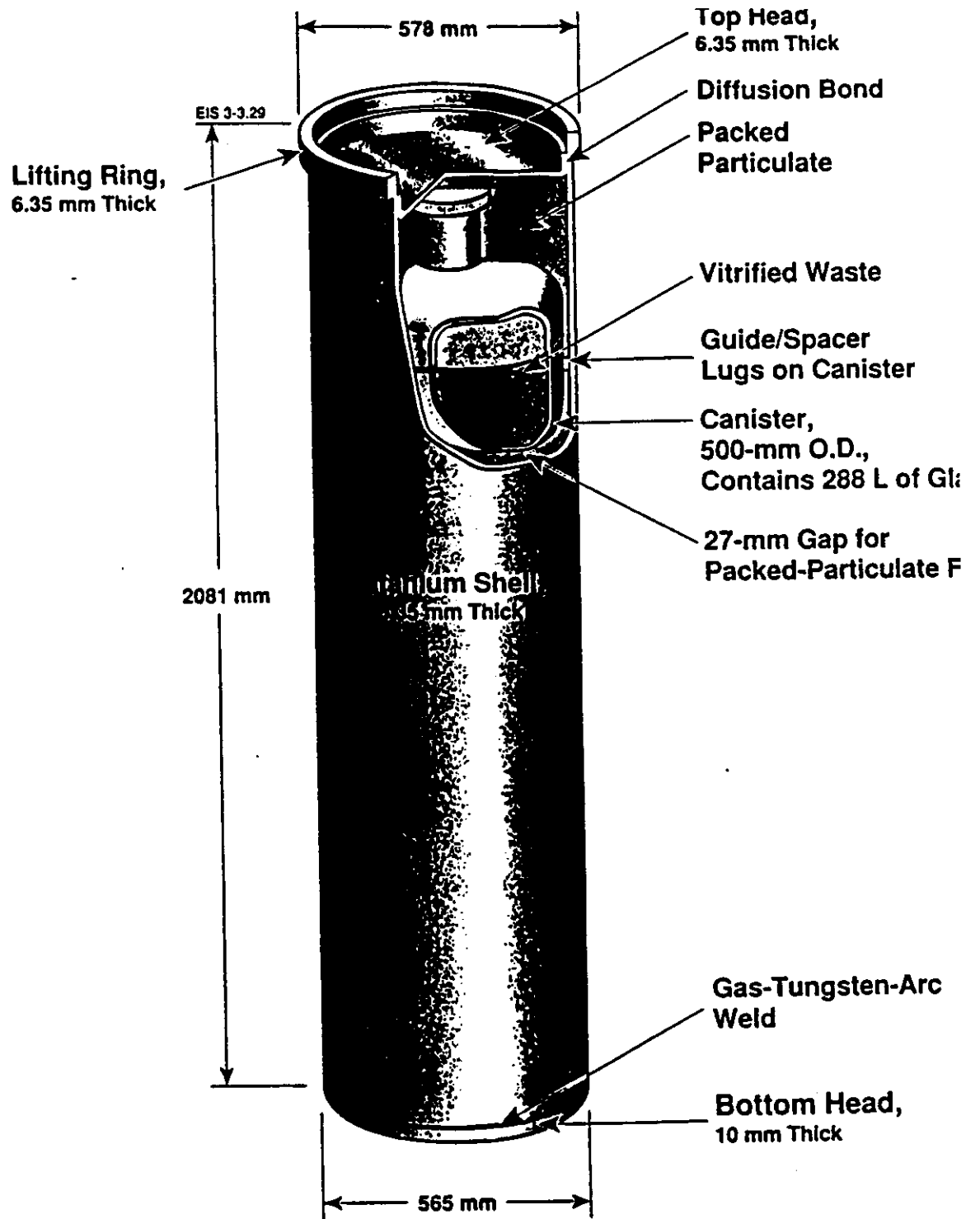
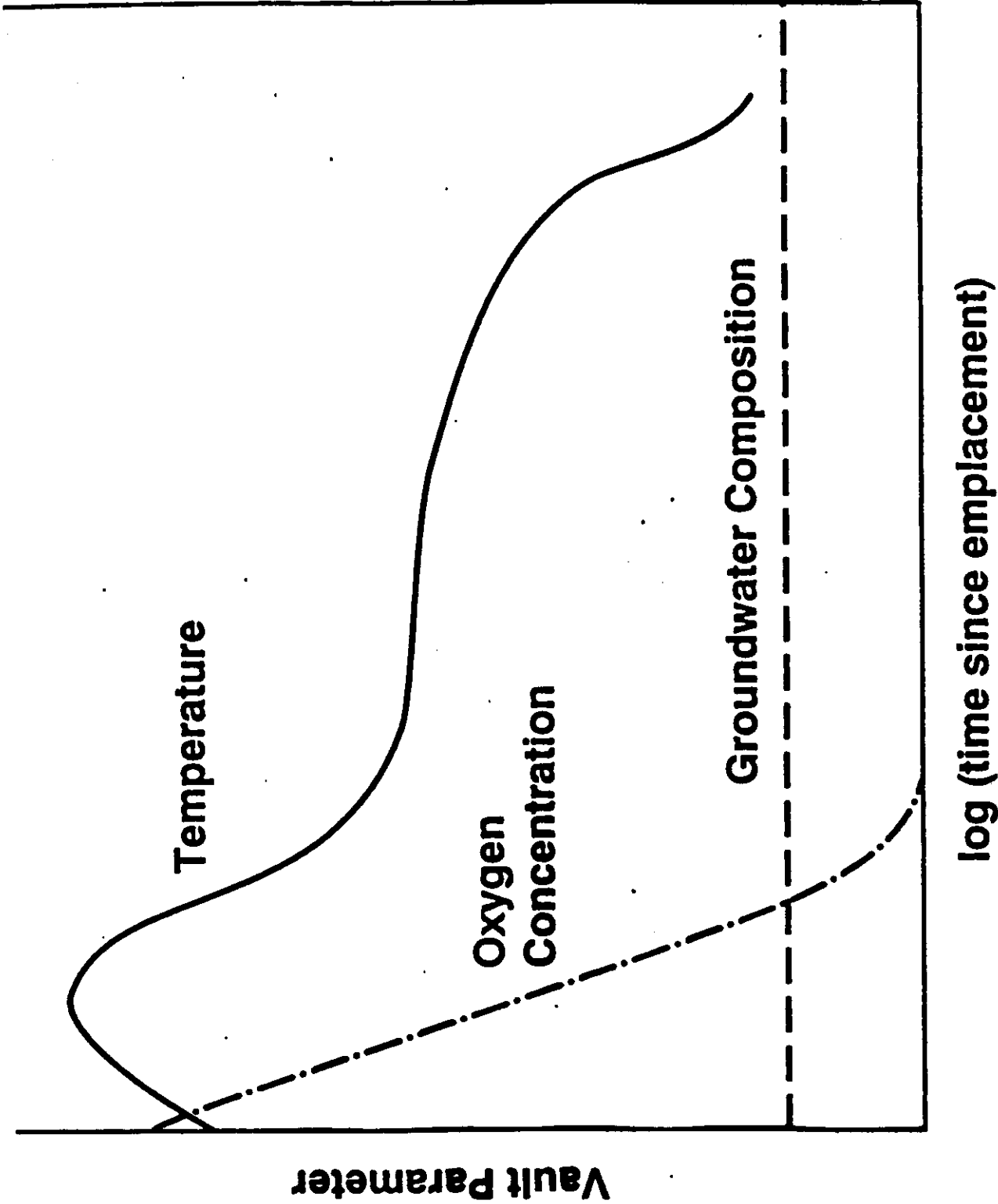
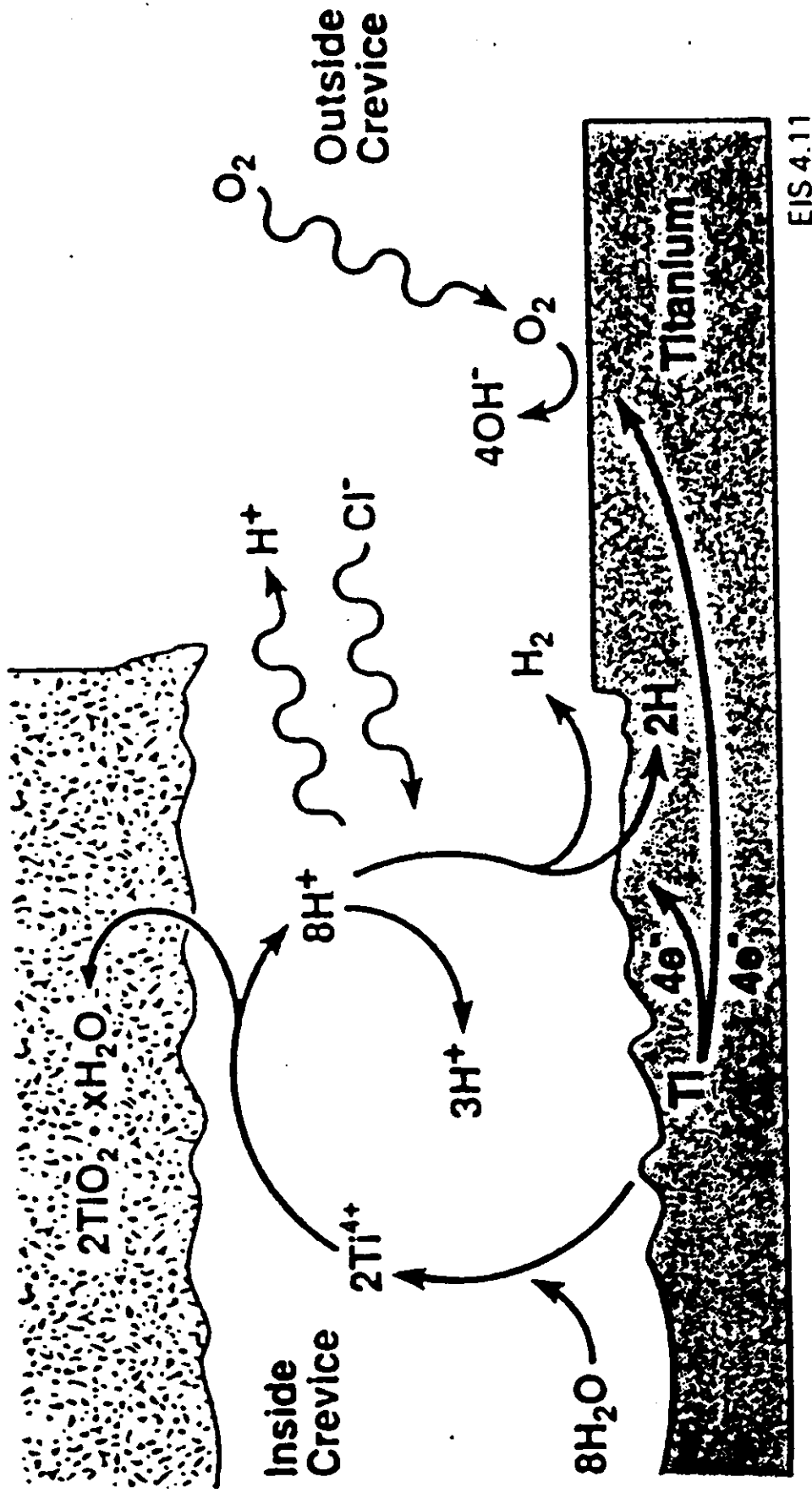


FIGURE 3-29: Titanium-Shell, Fuel-Reprocessing-Waste Disposal Cont with Vitrified-Waste Canister

CDG

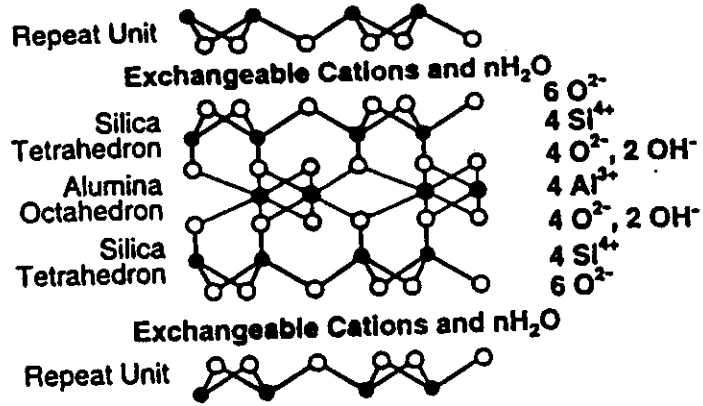


4-8a: Expected Changes in Those Vault Parameters That Would Affect



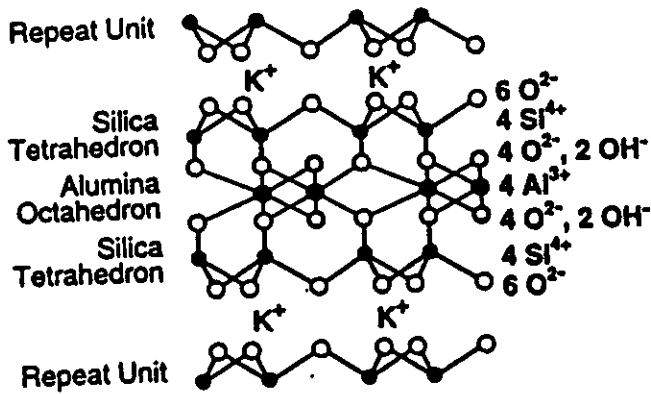
EIS 4.11

FIGURE 4-11: SCHEMATIC SHOWING THE BASIC ELECTROCHEMICAL, CHEMICAL, AND TRANSPORT STEPS INVOLVED IN THE CREVICE



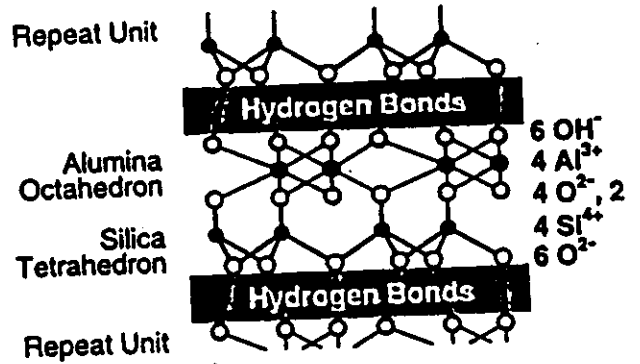
Montmorillonite Crystal Structure

Isomorphous substitutions and imperfections in the crystal lattice give high negative charge.
(Cation exchange capacity = 80 meq/100 g, specific surface area = 600 m²/g.)



Illite Crystal Structure

Structure is similar to montmorillonite; potassium ions bond the silica-alumina layers. Crystal size is greater and surface activity is less than that of montmorillonite.
(Cation exchange capacity = 20 meq/100 g, specific surface area = 80 m²/g.)



Kaolinite Crystal Structure

Kaolinite has the lowest surface activity and largest crystal size.
(Cation exchange capacity = 5 meq/100 g, specific surface area = 20 m²/g.)

FIGURE 4-3: Crystal Structures of Some Common Clay Minerals (after Lant and Whitman 1969)

BDBS

VDE

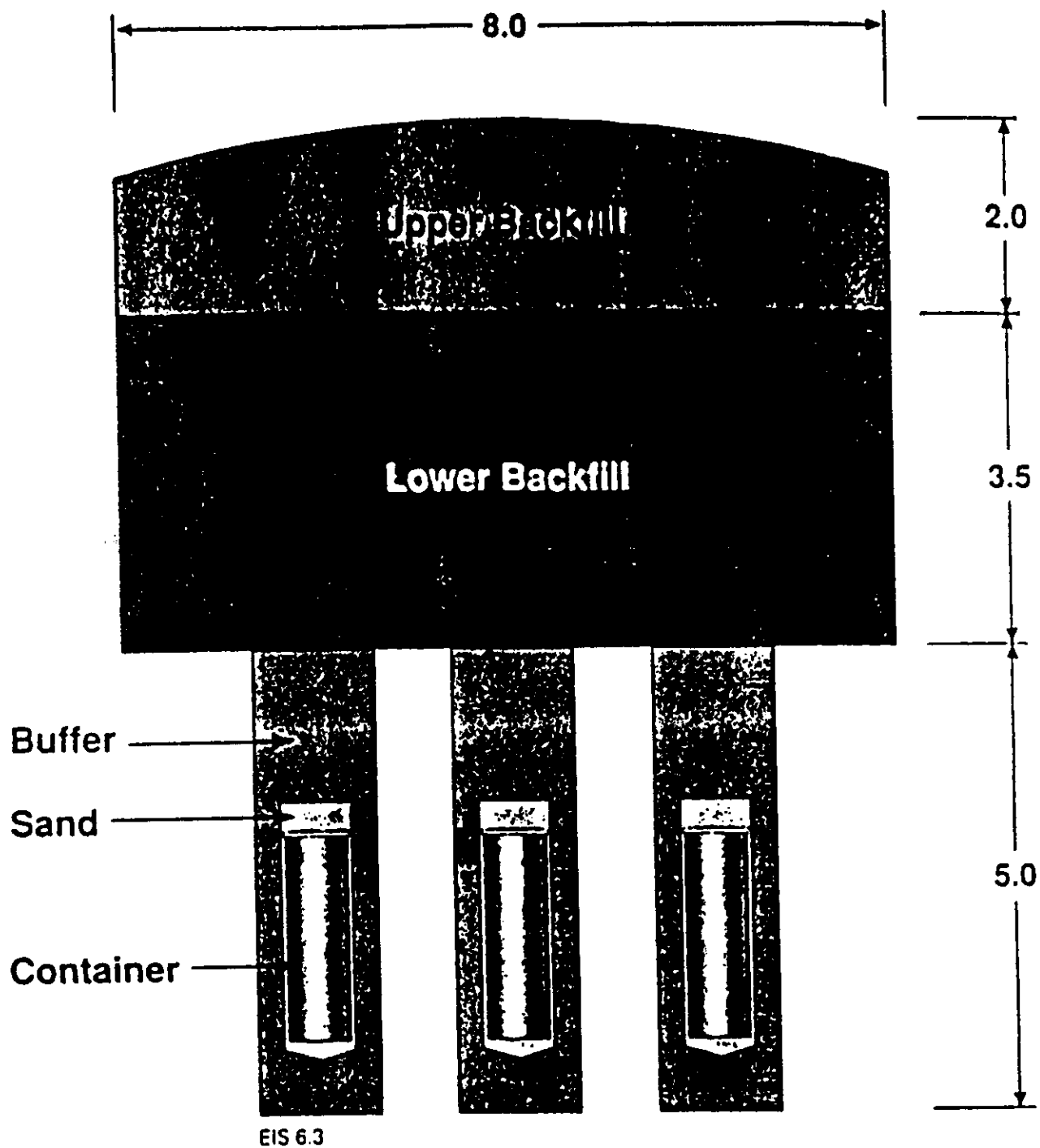


FIGURE 6-3: CROSS SECTION OF A FILLED DISPOSAL ROOM IN THE PRE-DISPOSAL-FACILITY

VDE

POSTCLOSURE ASSESSMENT

1. Specify System Features

2. Identify Scenarios

3. Develop Models & Data

4. Estimate Impacts

5. Analyze Sensitivity

6. Compare with Criteria

Environmental
Impact
Statement

Research & Development
Site Characterization and
Monitoring



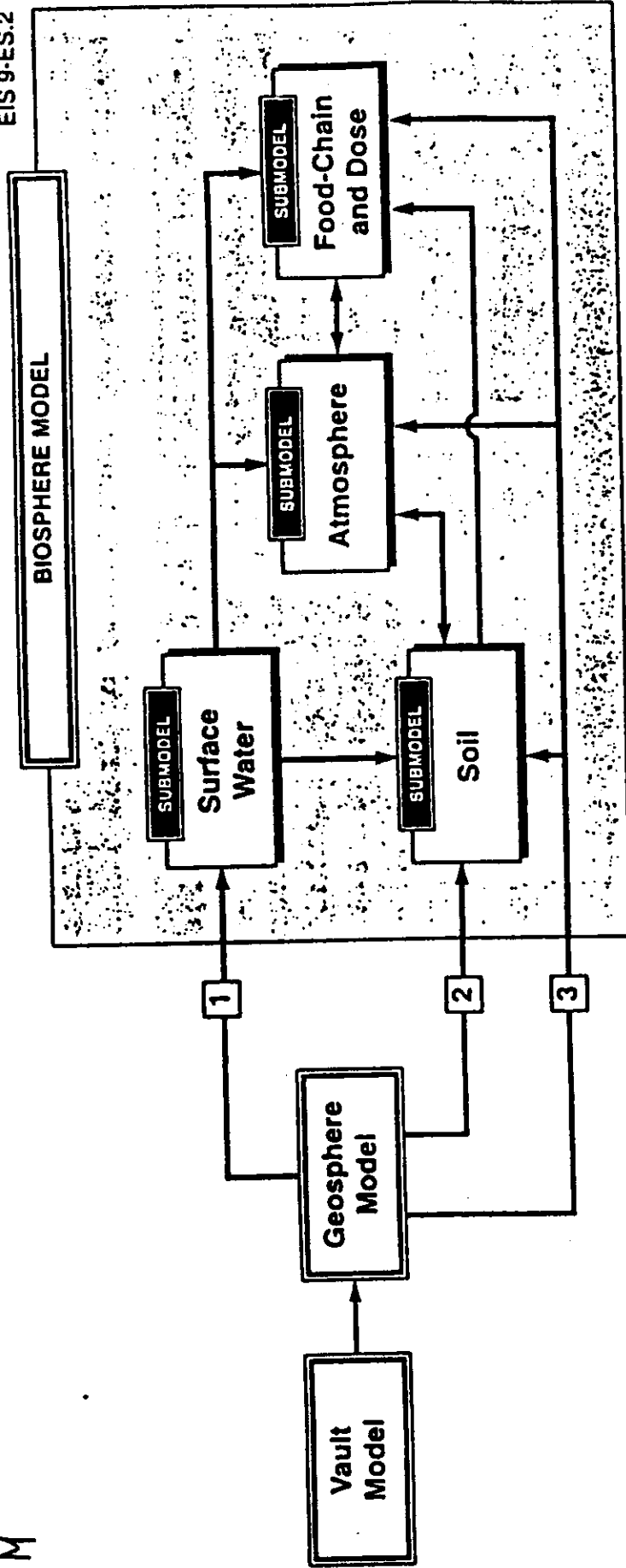
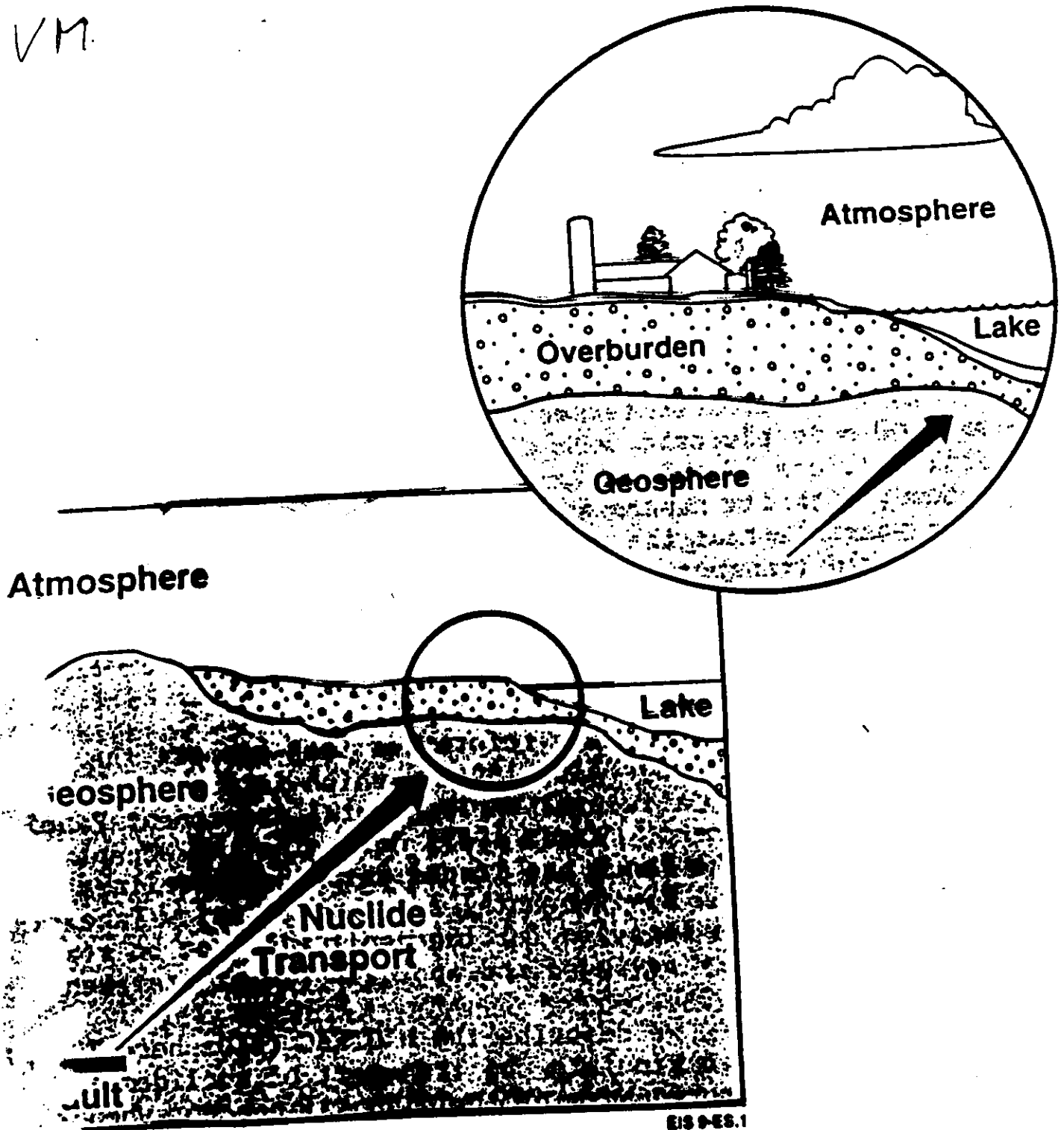


FIGURE ES-2: Schematic Representation of the Three Main Assessment Models (Vault, Geosphere and Biosphere) for the Disposal Concept Assessment, and of the Main Nuclide Transfers Among the Four Submodels of the Biosphere Model (Surface Water, Soil, Atmosphere, and Food-Chain and Dose) and Between the Geosphere and Biosphere Models. Discharges from the geosphere to the biosphere model are: (1) aquatic, (2) terrestrial and (3) well.

VM



Schematic Representation of Groundwater Transport of Nuclides from the Vault, 500 to 1000 m Underground, to the Biosphere (Enlarged Insert)

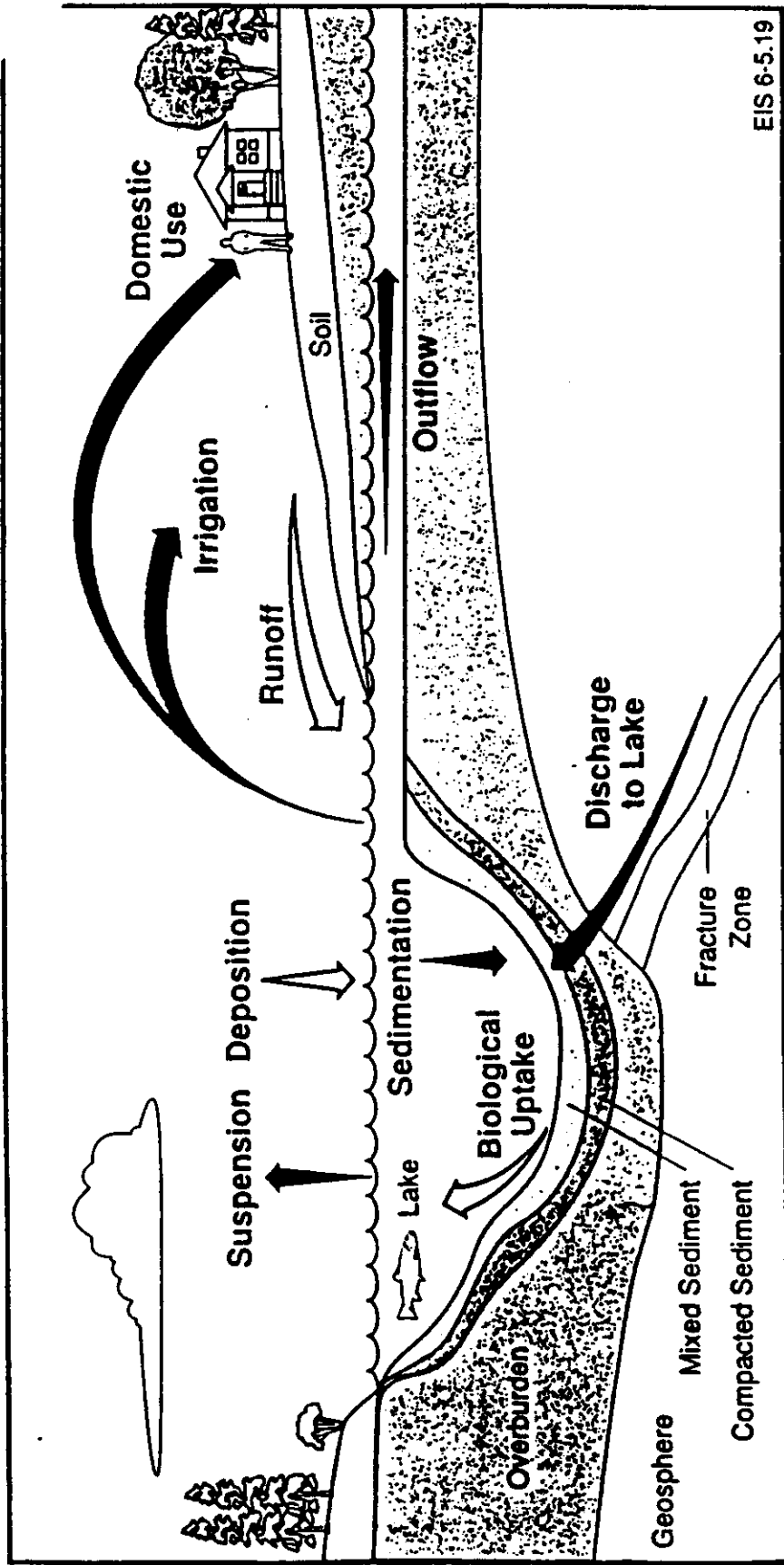


FIGURE 5-19: The Processes Modelled in the Lake and Lake Sediments Model

The solid arrows show processes that are explicitly modelled in the surface water model and the open arrows indicate processes that are implicitly considered (Davis et al. 1993). The primary sources of contaminants are the well (not shown) and groundwater discharging through the overburden to the compacted sediments (these sediments underlie Pinawa Channel and Bogy Creek shown in Figure 5-16). Contaminants leave the lake water by radioactive decay (not illustrated), particle suspension and degassing to the atmosphere, sedimentation to the mixed sediments, pumping for domestic and irrigation use by the critical group, and outflow downstream. We assume that all contaminants eventually return to the lake, except those lost by radioactive decay, outflow, and (for ^{14}C and the noble gases) by degassing.

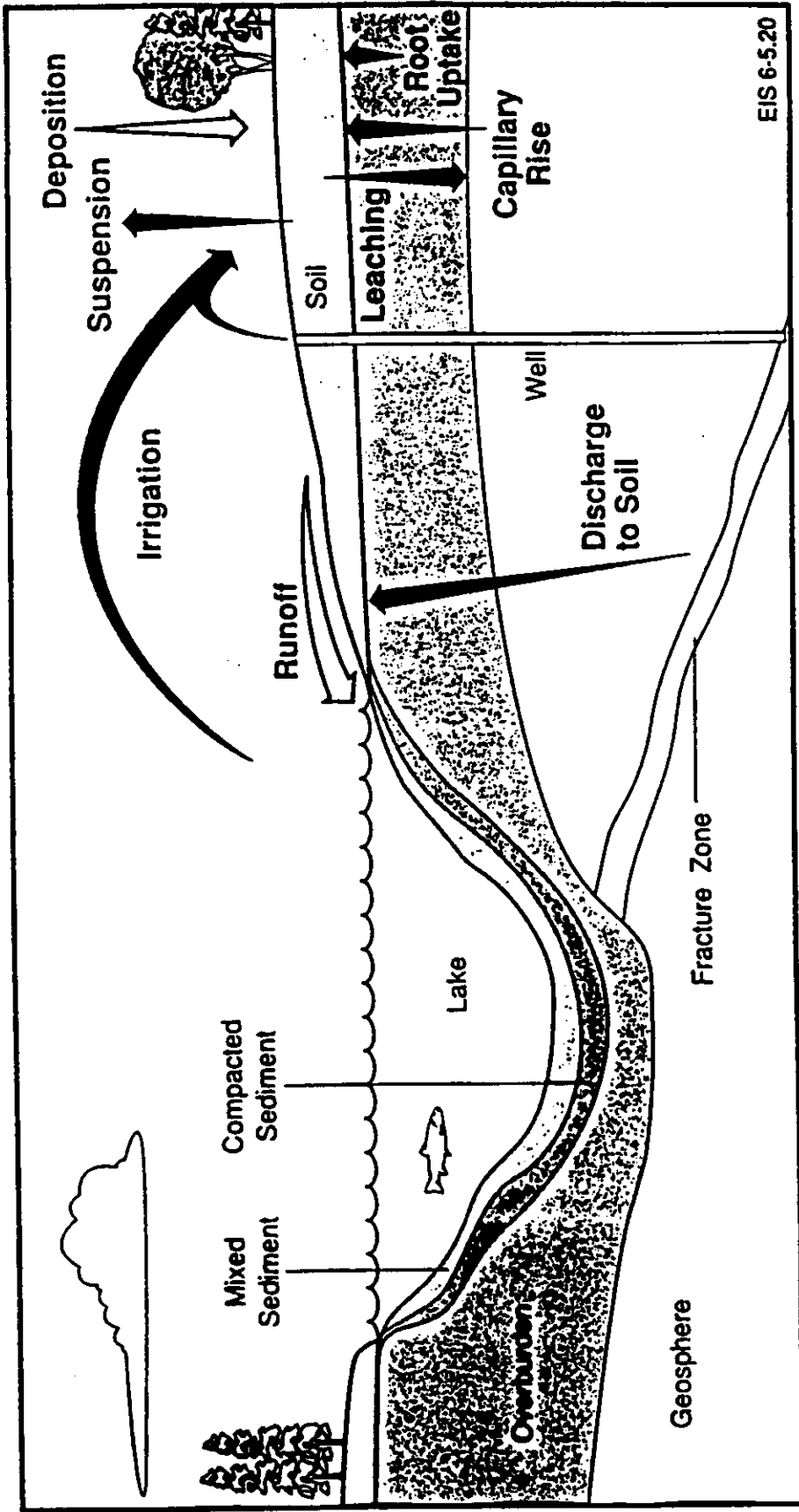
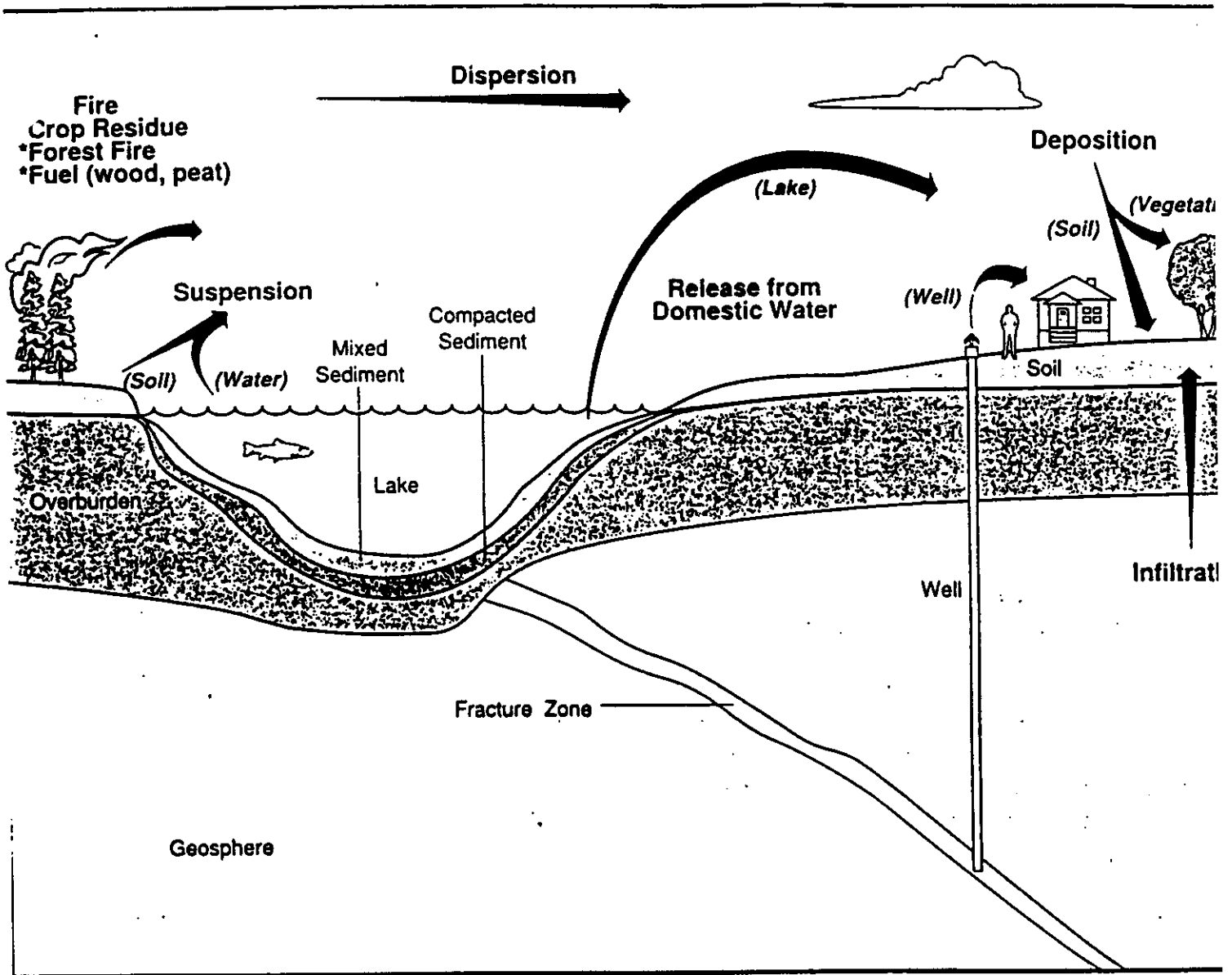


FIGURE 5-20: The Processes Modelled in the Soil Model

The solid arrows show processes that are explicitly modelled in the soil model, and the open arrows indicate processes that are implicitly considered (Davis et al. 1993). Four fields are modelled in a similar manner: a garden, a forage field, a woodlot and a peat bog (shown in Figure 5-16). Contaminants enter each field by capillary rise from the water table below the soil, by air deposition of contaminants, and (for the garden and forage field only) by irrigation using water from the lake or well. Contaminants leave each area by leaching, suspension, root uptake, and runoff to the lake.

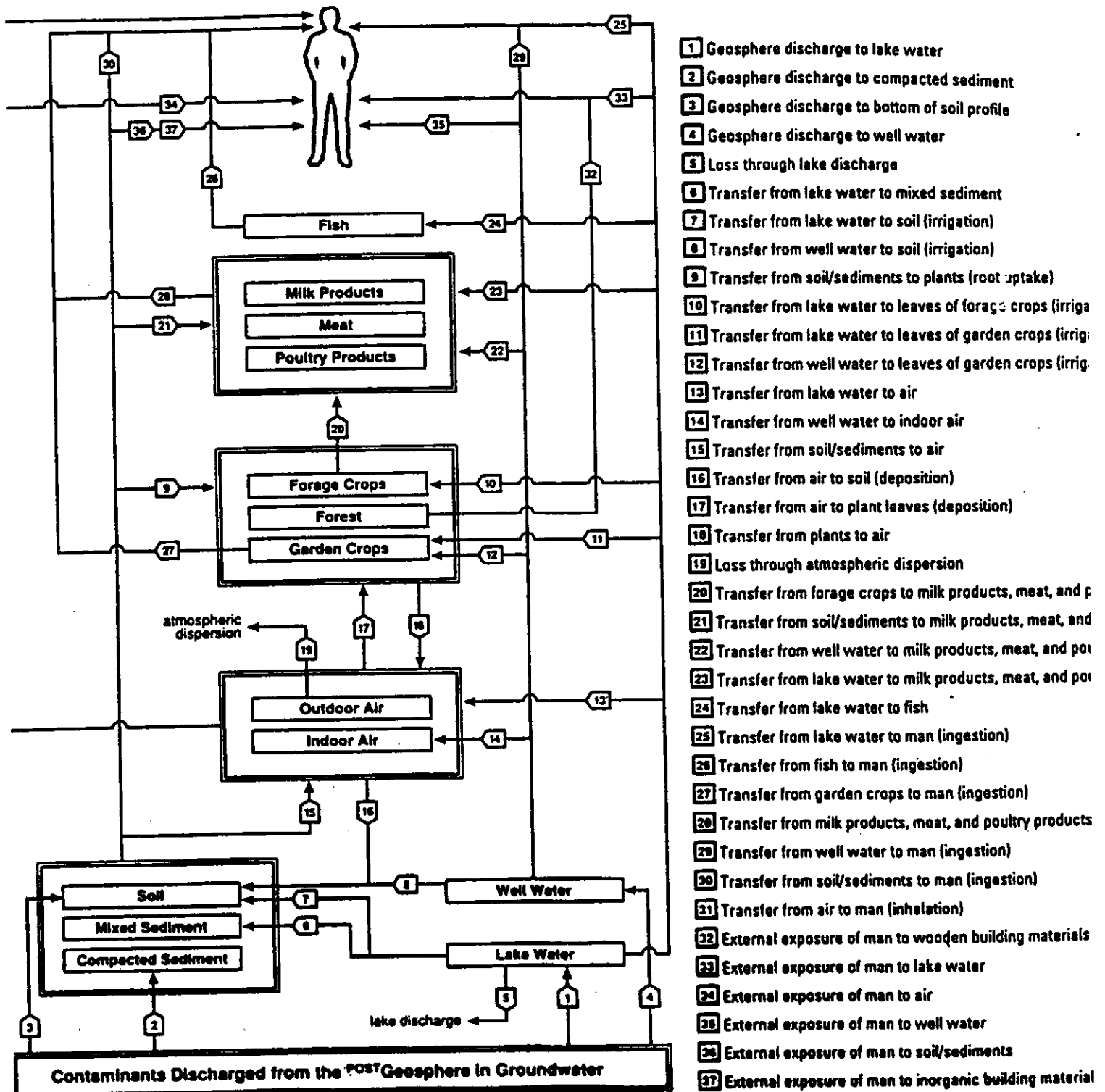


EIS

HMA

FIGURE 5-21: The Processes Modelled in the Atmosphere Model

The arrows show processes that are explicitly modelled in the atmosphere model (Davis et al. 1996). Contaminants enter outside air by degassing and suspension of particulates from the soil in the fields, from the water of the lake and from fires (including burning wood and peat for fuel). Contaminants enter dwellings with the outside air, by releases from domestic water (from the lake and the well), and by infiltration from soil around building foundations. We assume that the contaminants are well mixed by dispersion in the air.



EIS 7.5

FIGURE 7-5: COMPARTMENTS AND PATHWAYS IN THE POST BIOSPHERE-MODEL

AMB

HA

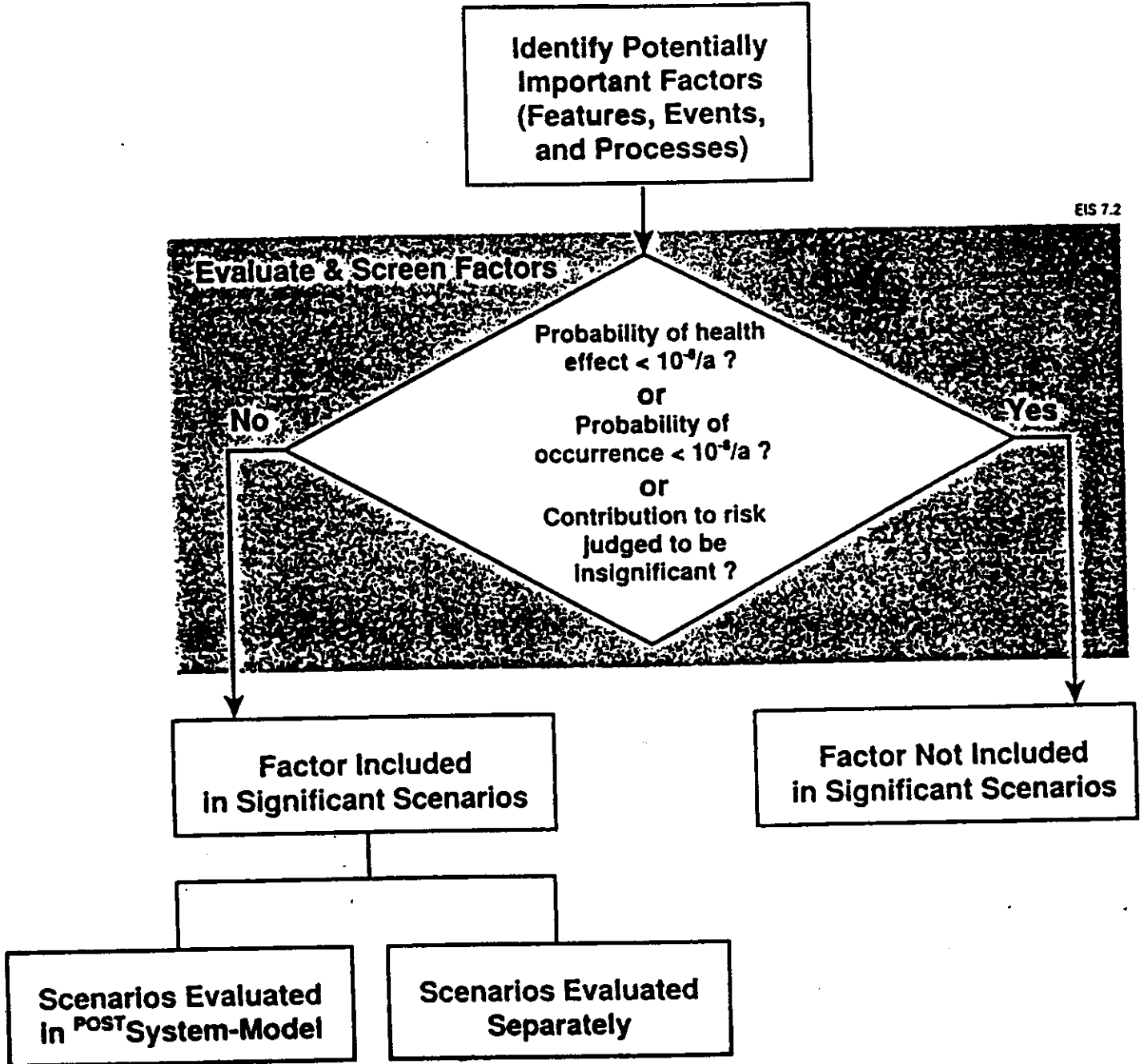
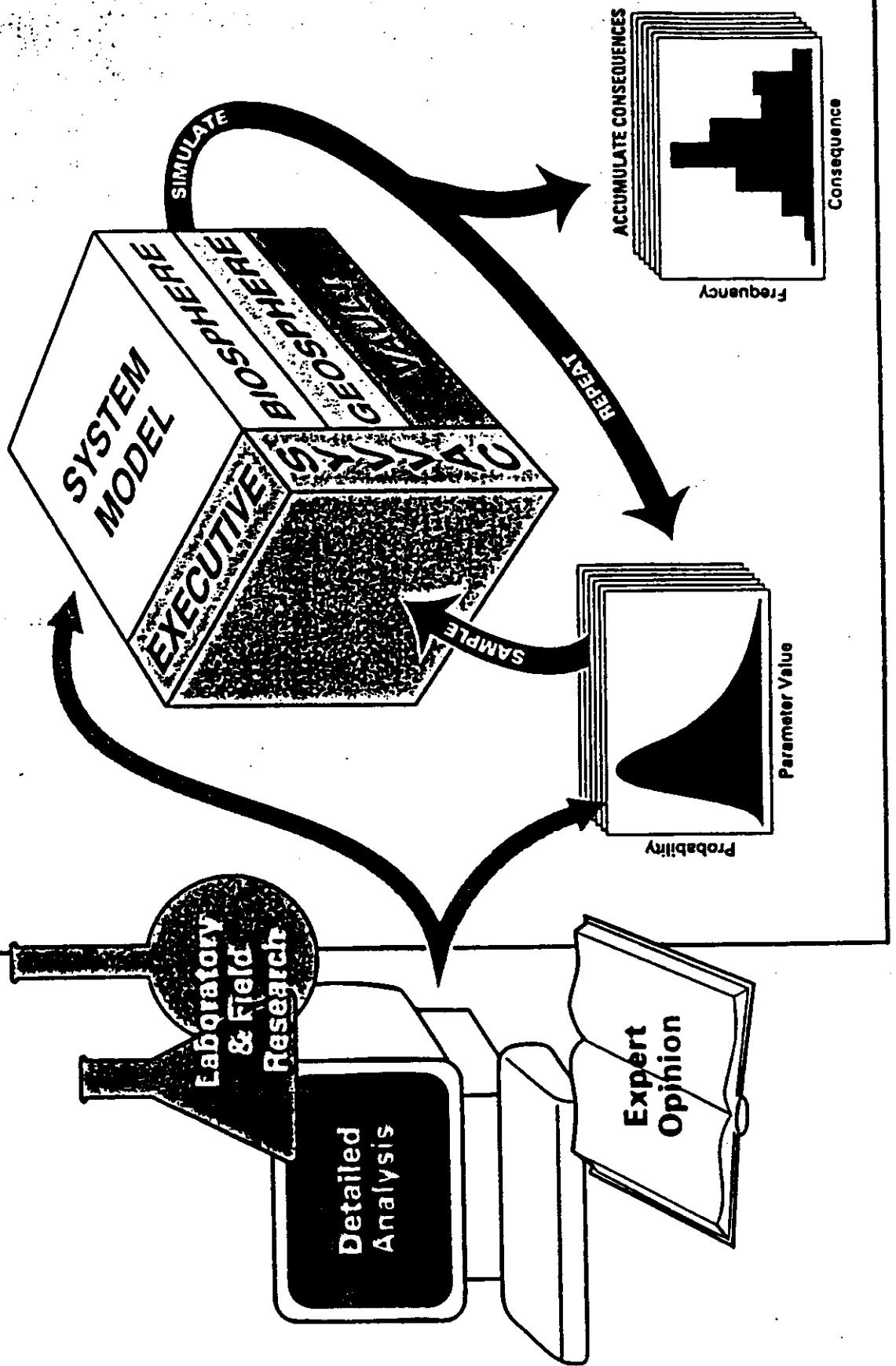


FIGURE 7-2: PROCESS FOR IDENTIFYING AND EVALUATING SIGNIFICANT SCENAR

SYSTEMS VARIABILITY ANALYSIS



DDB

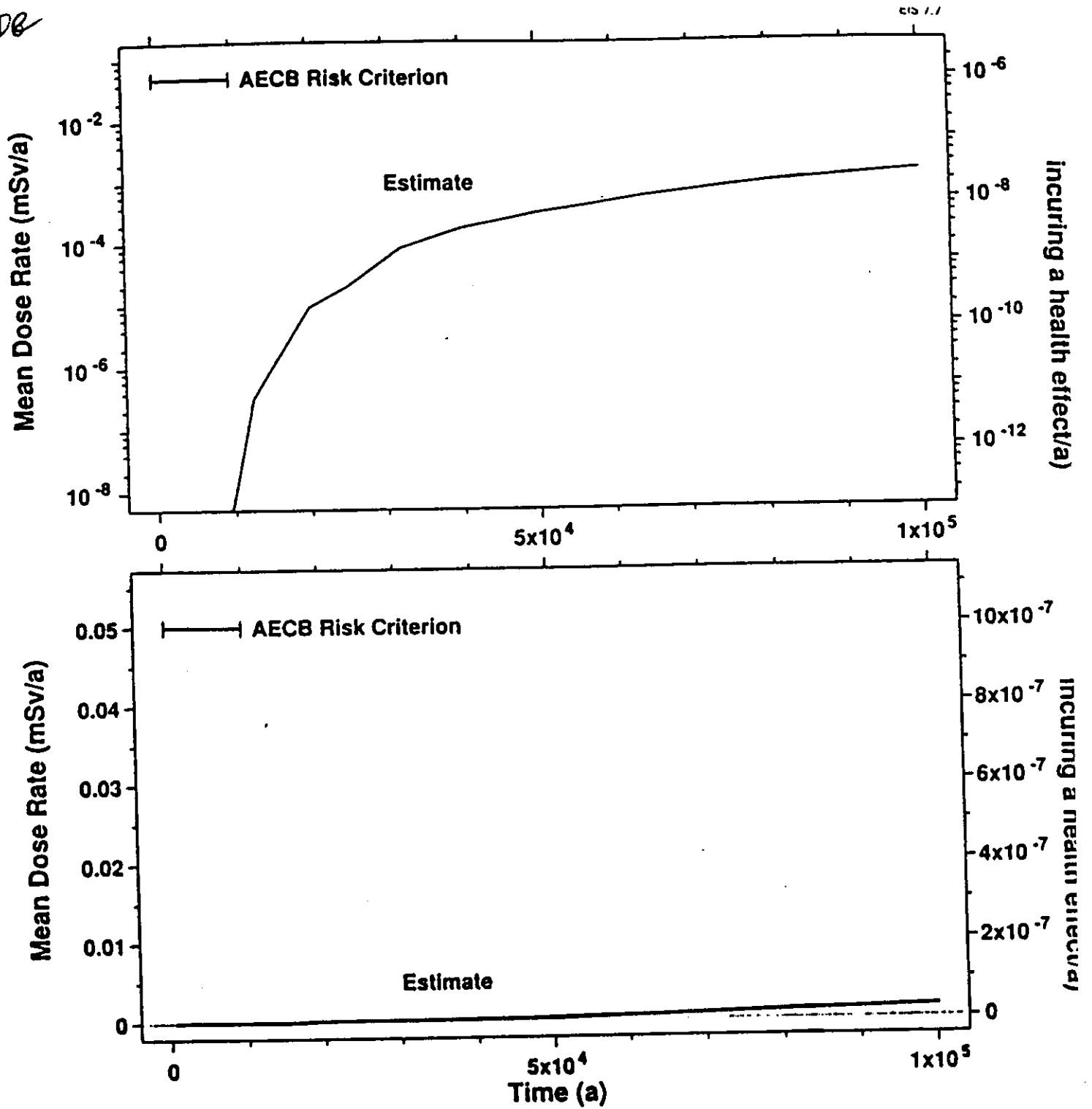


FIGURE 7-7: ESTIMATED MEAN DOSE RATE AND RISK AS A FUNCTION OF TIME

DDB

DDAF

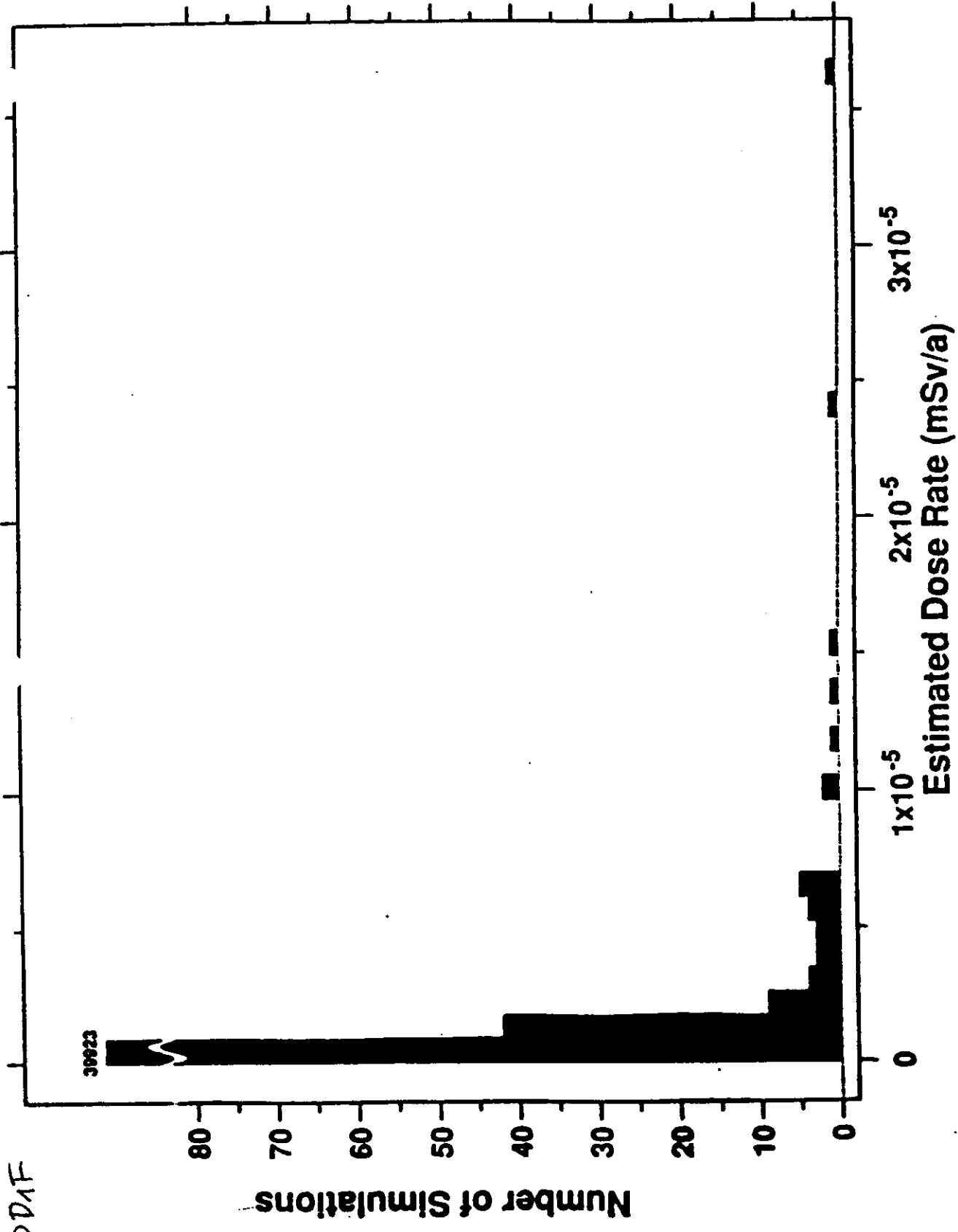


FIGURE 7-8: ESTIMATED DOSE RATES AT 10 000 YEARS

DD

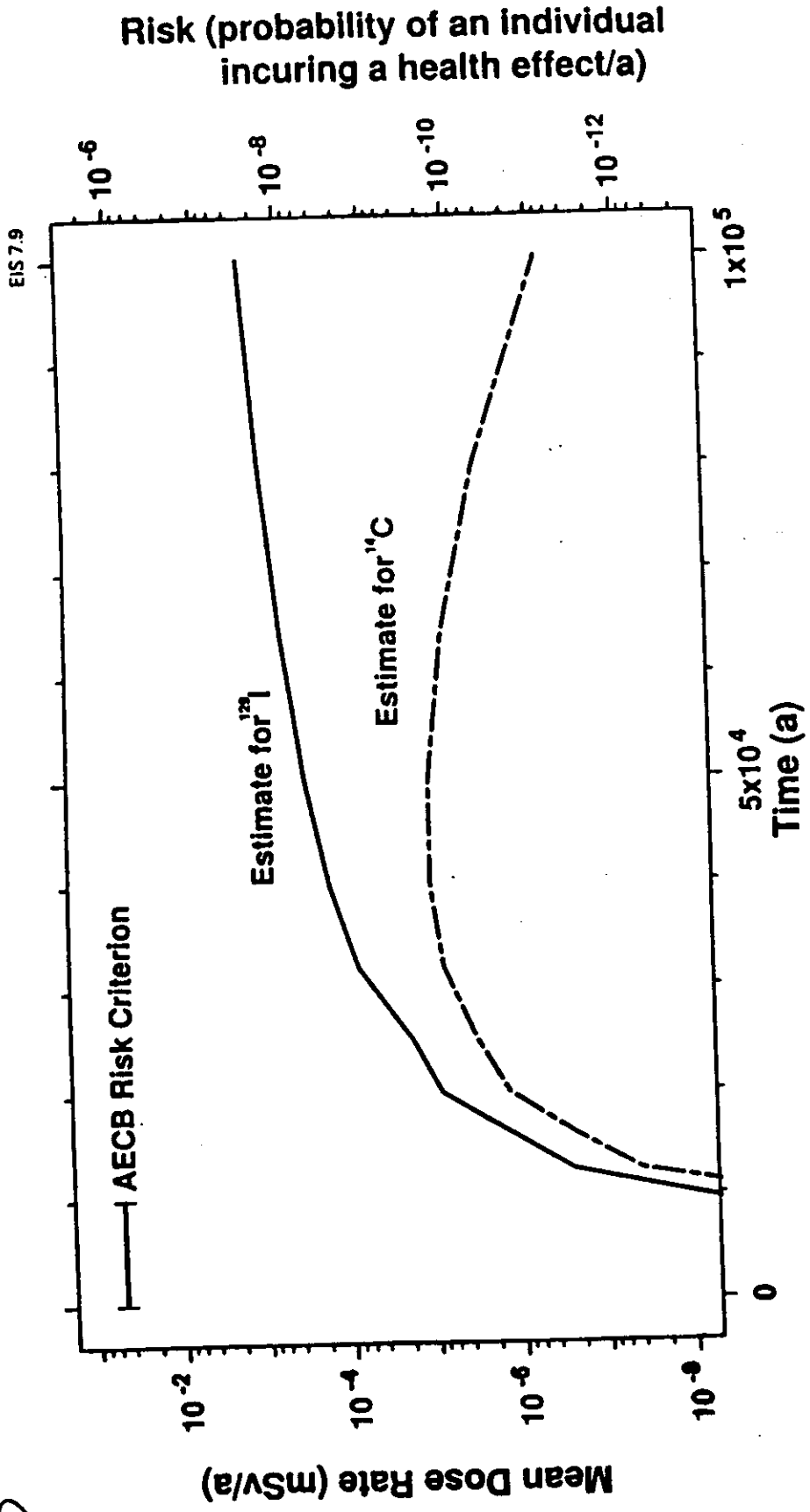
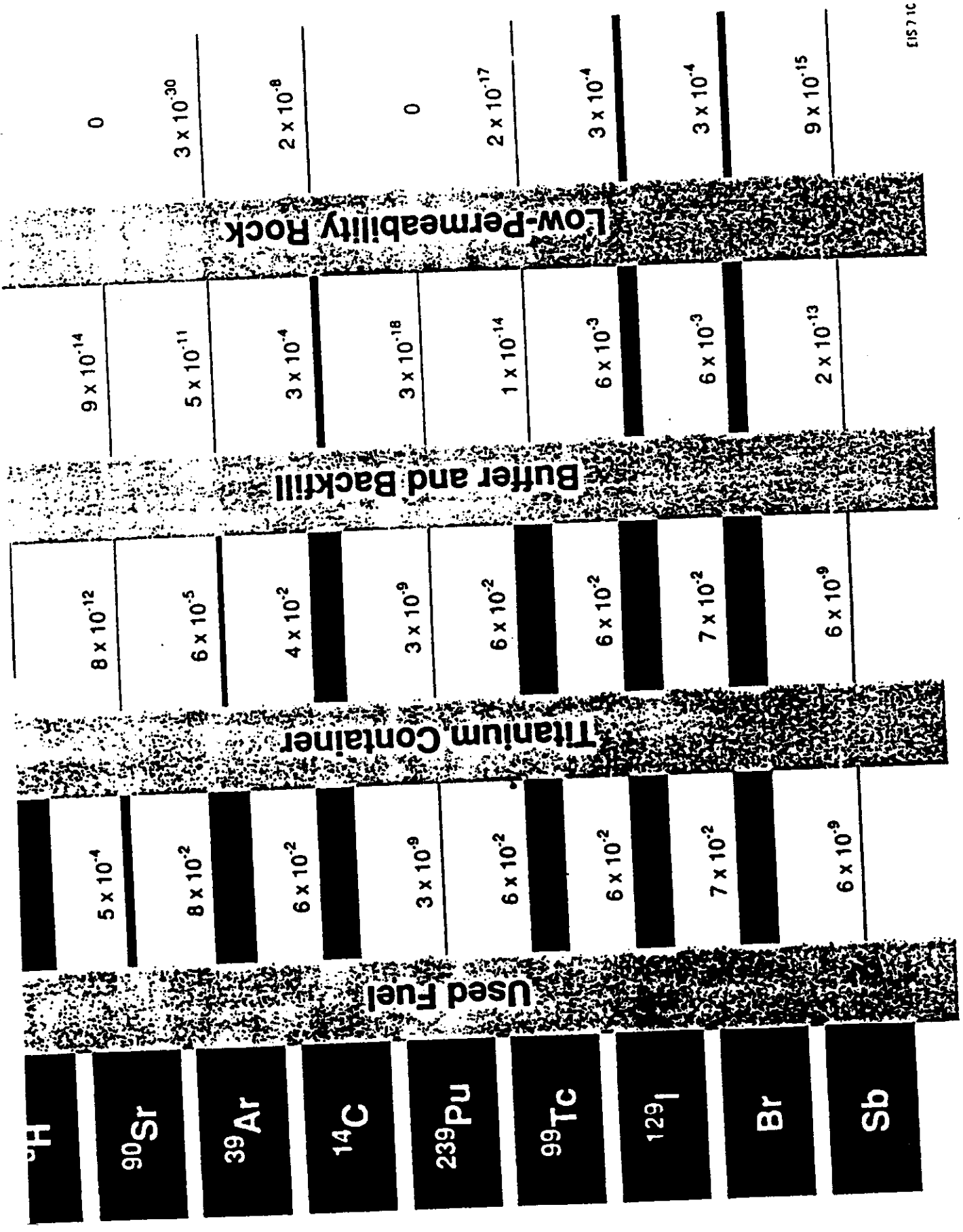


FIGURE 7-9: CONTRIBUTIONS OF ^{129}I AND ^{14}C TO THE ESTIMATED MEAN DOSE RATE AND RISK



ES 71C

FIGURE 7-10: CUMULATIVE FRACTION OF A NUCLIDE RELEASED BY THE BARRIERS