

# **ACR Moderator Circulation**

**By Dave Wren, ACR R&D Manager, ACR Development Project**

**Presented to US Nuclear Regulatory Commission**

**Office of Nuclear Reactor Regulation**

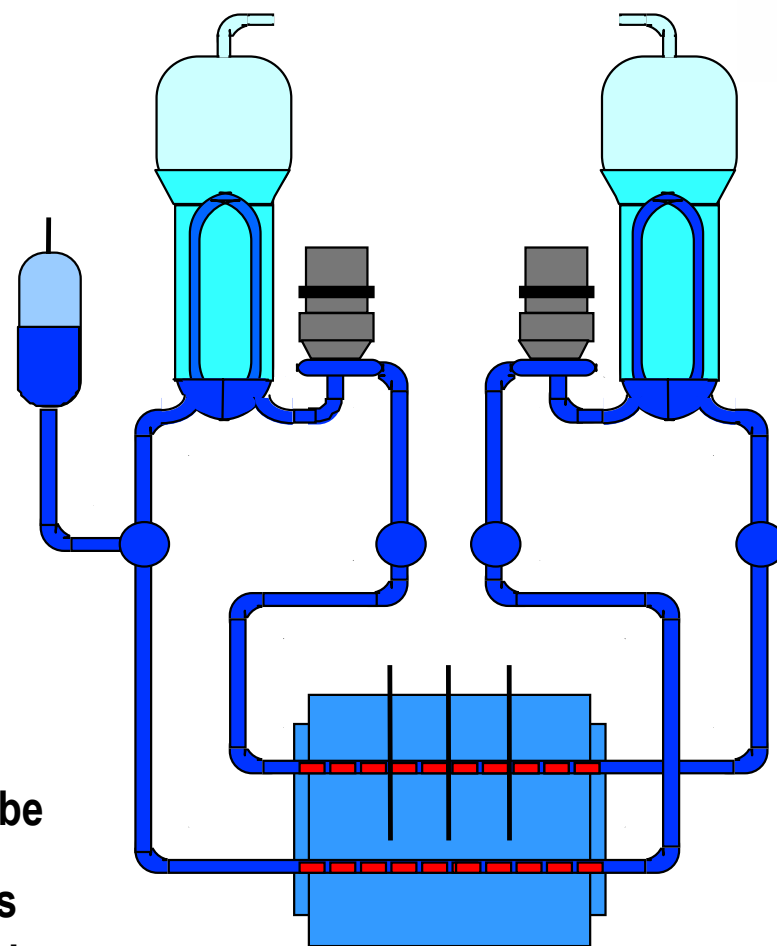
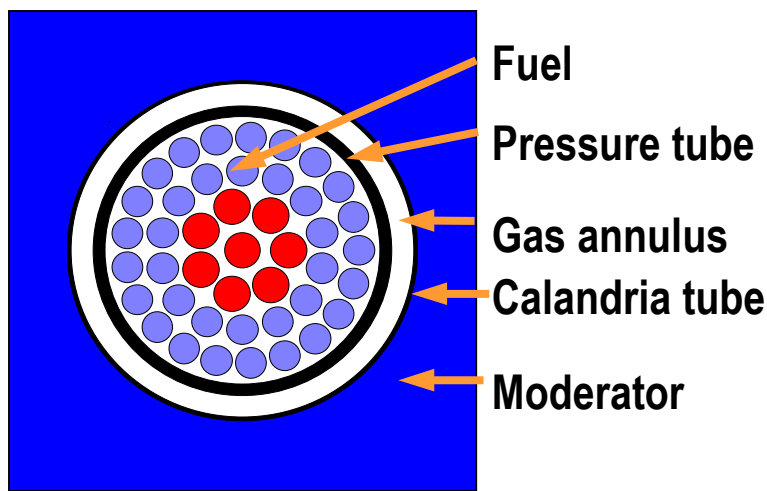
**September 26, 2002**





# CANDU Moderator

- The moderator is a low-pressure, low temperature system (separate from the reactor coolant).
- Heat is continuously removed from the moderator during normal operation.
- The moderator acts as a heat sink in the event of certain accident sequences





# Moderator Thermalhydraulics

- **Need to predict moderator circulation patterns and temperature distribution to ensure adequate cooling margin for all channels.**
- **Need to predict moderator circulation for postulated accident scenarios to ensure availability of moderator heat sink.**
- **A 3-D single phase computational fluid dynamics computer code (MODTURC\_CLAS) is used to predict moderator flow and temperature distribution.**



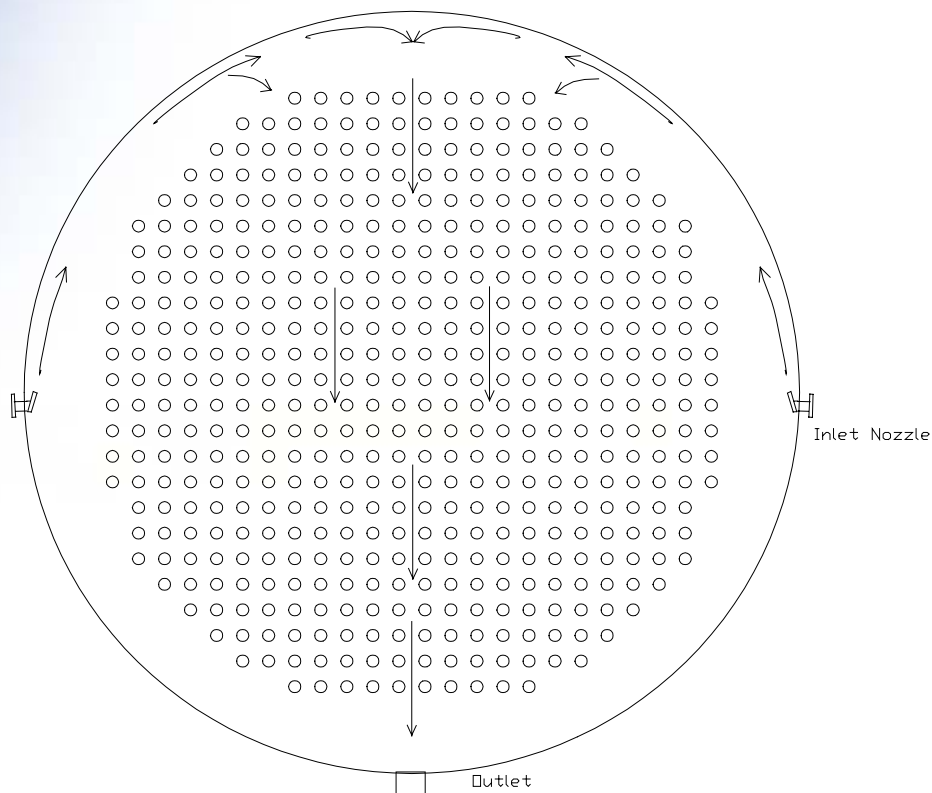
# **MODTURC\_CLAS Validation Base**

- **Database for operating CANDU reactors**
  - Data from a 2D facility (1/4-scale)
  - In-reactor temperature measurements
  - Difficulties in validating flow conditions
- **A more efficient circulation system was developed and validated for CANDU 9.**
- **A similar design will be used for ACR.**

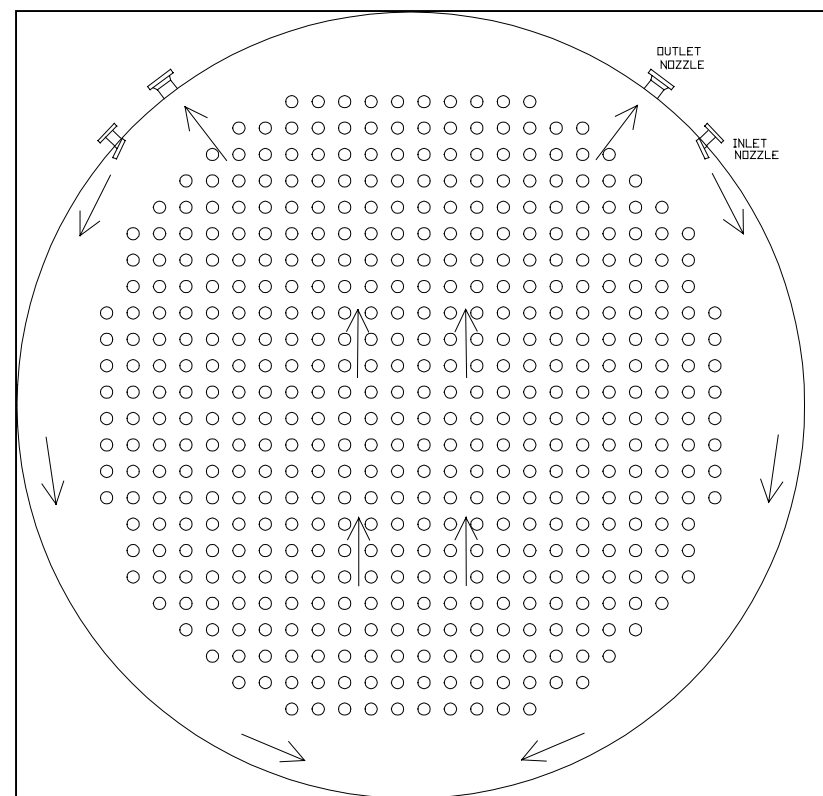


# Moderator Circulation Design

## CANDU 6



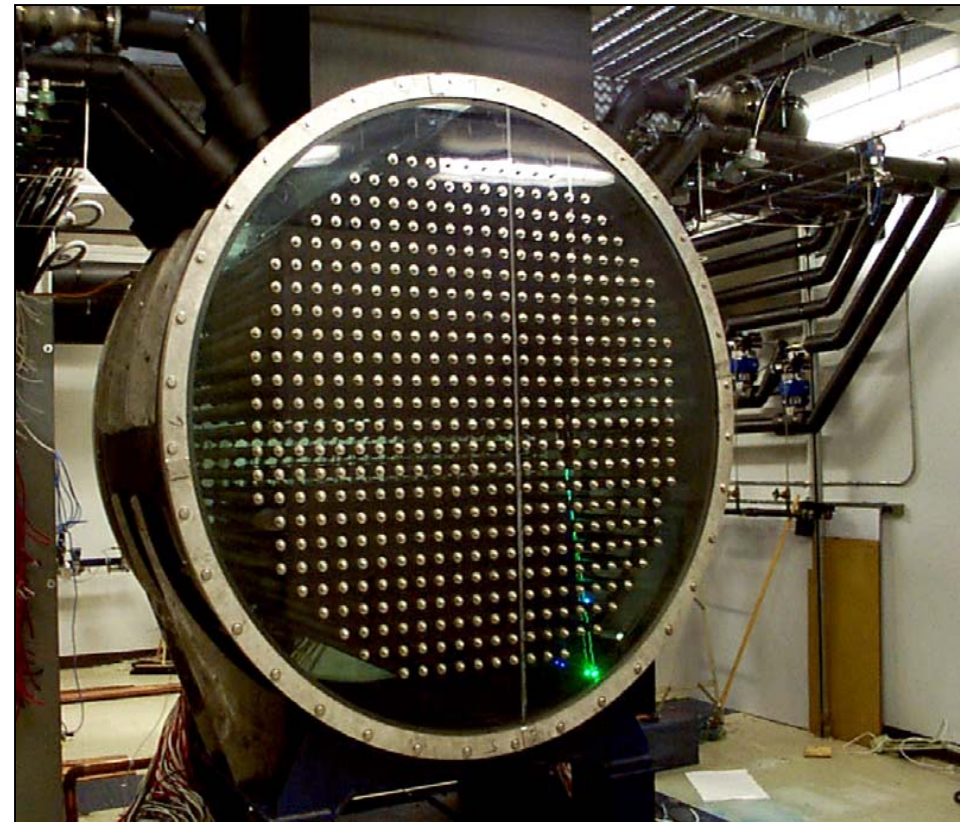
## CANDU 9 and ACR





# Moderator Test Facility

- Large-scale facility to measure three-dimensional velocity and temperature distributions in moderator geometry
- Calandria vessel with fuel channel simulator heaters and coolant flow system
- 1/4 scale calandria used to validate CANDU 9 design
- 1/3 scale calandria is being built to validate ACR design





# Moderator Test Facility Scaling

- **Scale was decided based on analysis of the relevant equations (mass, momentum, and energy).**
- **The following dimensionless numbers were identified:**
  - Archimedis Number ( $Ar$ )
  - Dimensionless volumetric heat source
  - Prandtl Number ( $Pr$ )
  - Reynolds Number ( $Re$ )
- **It was shown that only the first three dimensionless numbers needed to be matched, provided the flow was turbulent (verified in tests).**



# Moderator Test Facility

<b>Parameter</b>	<b>CANDU 9</b>	<b>ACR</b>
<b>Number of heaters</b>	<b>480</b>	<b>284</b>
<b>Calandria diameter</b>	<b>2 m (6.6 ft)</b>	<b>1.3 m (4.3 ft)</b>
<b>Calandria length</b>	<b>1.5 m (4.9 ft)</b>	<b>1.9 m (6.2 ft)</b>
<b>Heater diameter</b>	<b>3.3 cm (1.30 in)</b>	<b>5.2 cm (2.05 in)</b>
<b>Fuel channel pitch</b>	<b>7.2 cm (2.83 in)</b>	<b>7.3 cm (2.87 in)</b>
<b>Maximum Power</b>	<b>1.7 MW</b>	<b>1.7 MW</b>



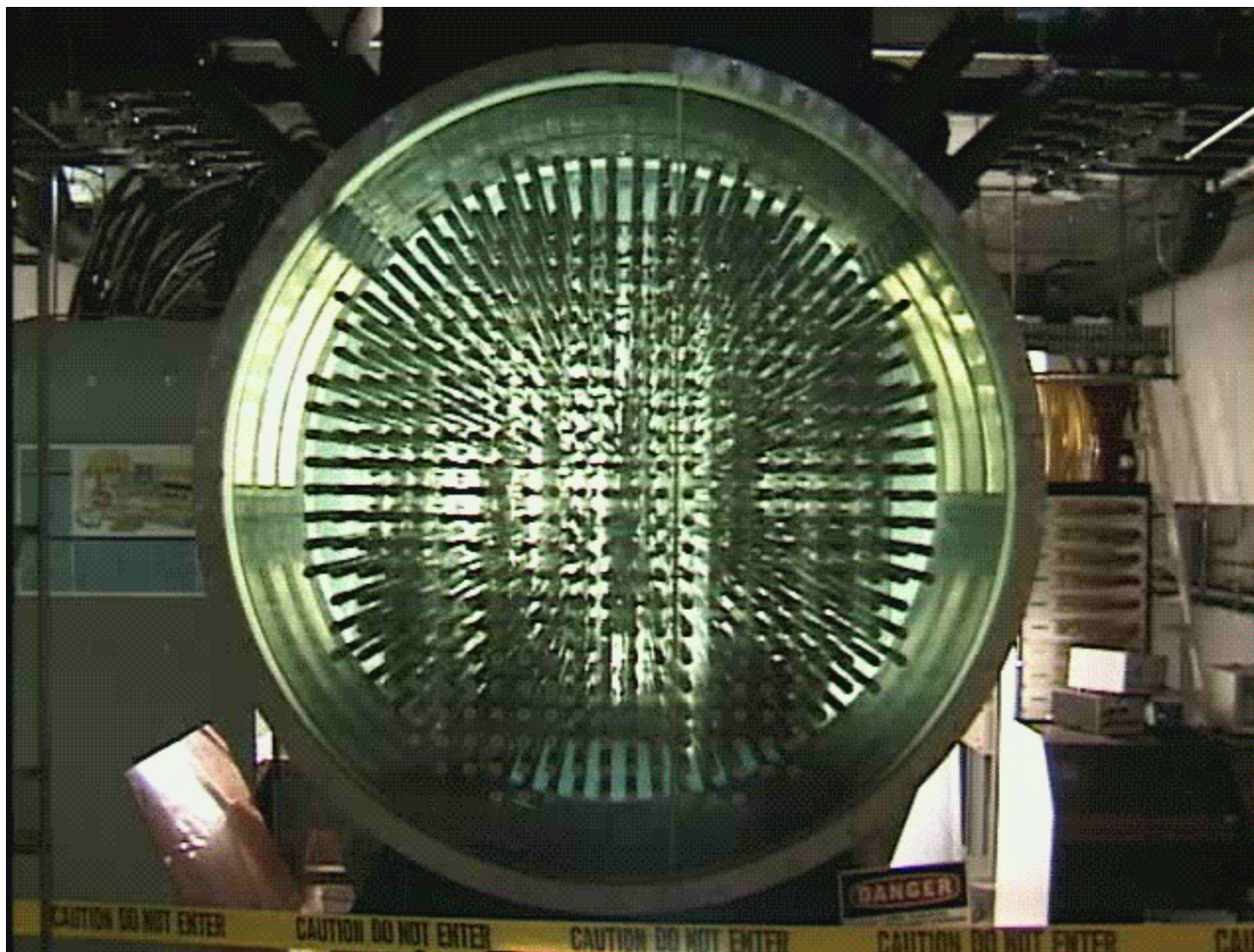


# Moderator Test Facility Capabilities

- **Typical measurements in MTF (steady-state tests)**
  - Flow visualization (using dye injection at the inlet nozzles).
  - 3-component velocity measurements using 3-D Laser Doppler Velocimetry (LDV).
  - Temperature distribution measurements using arrays of fixed and movable thermocouples.
  - A typical steady-state test lasts for 4 weeks.
- **Transient tests can also be performed (temperature measurements only) and last for about a day.**
- **Separate effects tests can also be performed to provide additional measurements to validate models used in MODTURC\_CLAS (pressure drop, jet development, buoyancy, etc).**

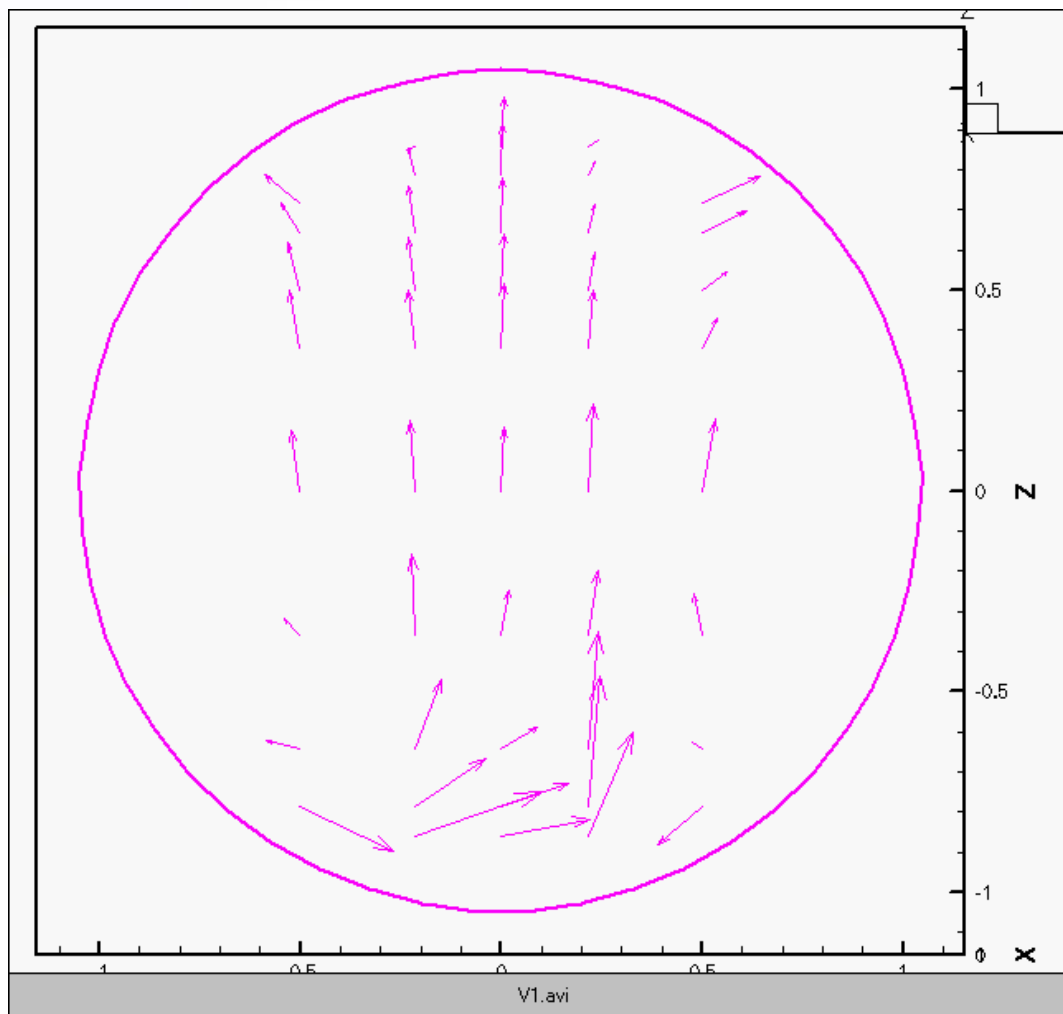


# Dye Injection Test



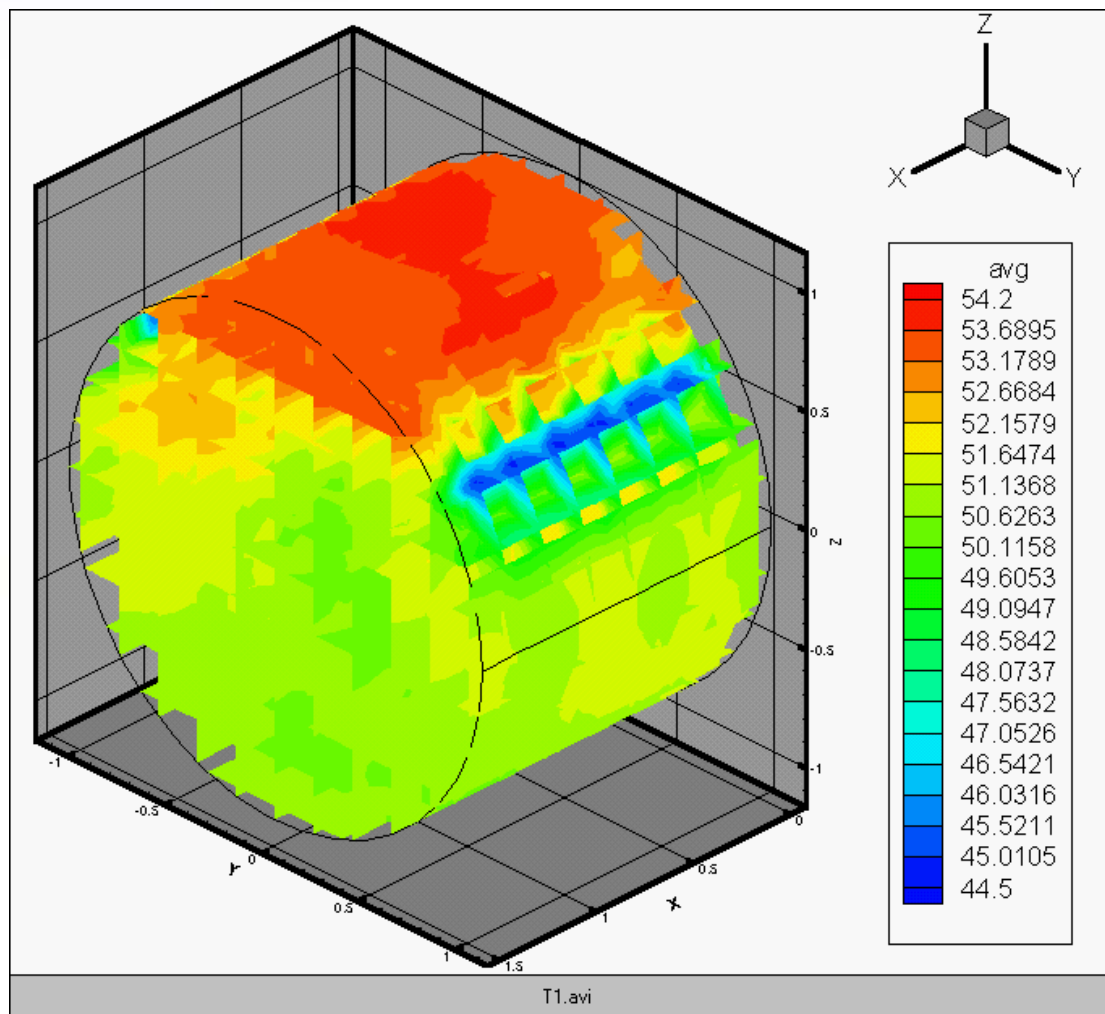


# CANDU 9 Flow Velocity Measurements





# CANDU 9 Thermal Measurements





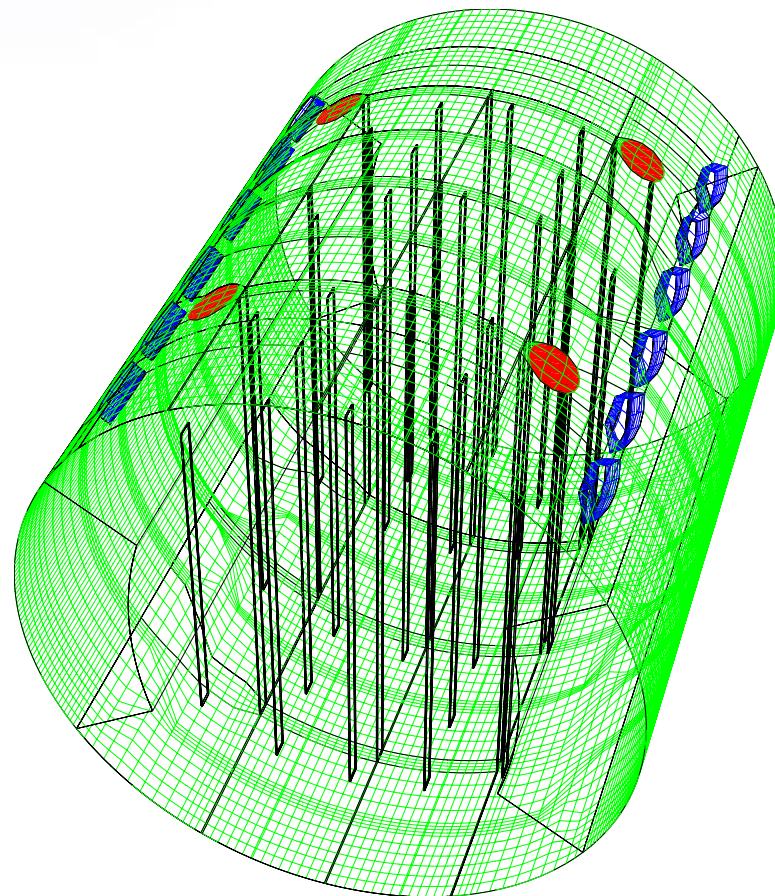
# ACR Moderator Circulation Design

- **Key ACR design aspects that affect moderator circulation:**
  - **Smaller lattice pitch (higher hydraulic resistance).**
  - **Reactivity devices (could affect local temperatures).**
- **Assessment of ACR preliminary design using MODTURC\_CLAS showed similar thermalhydraulic behavior to the CANDU 9, despite the smaller lattice pitch and the reactivity devices.**
- **Tests in a modified MTF will be conducted to verify the design.**





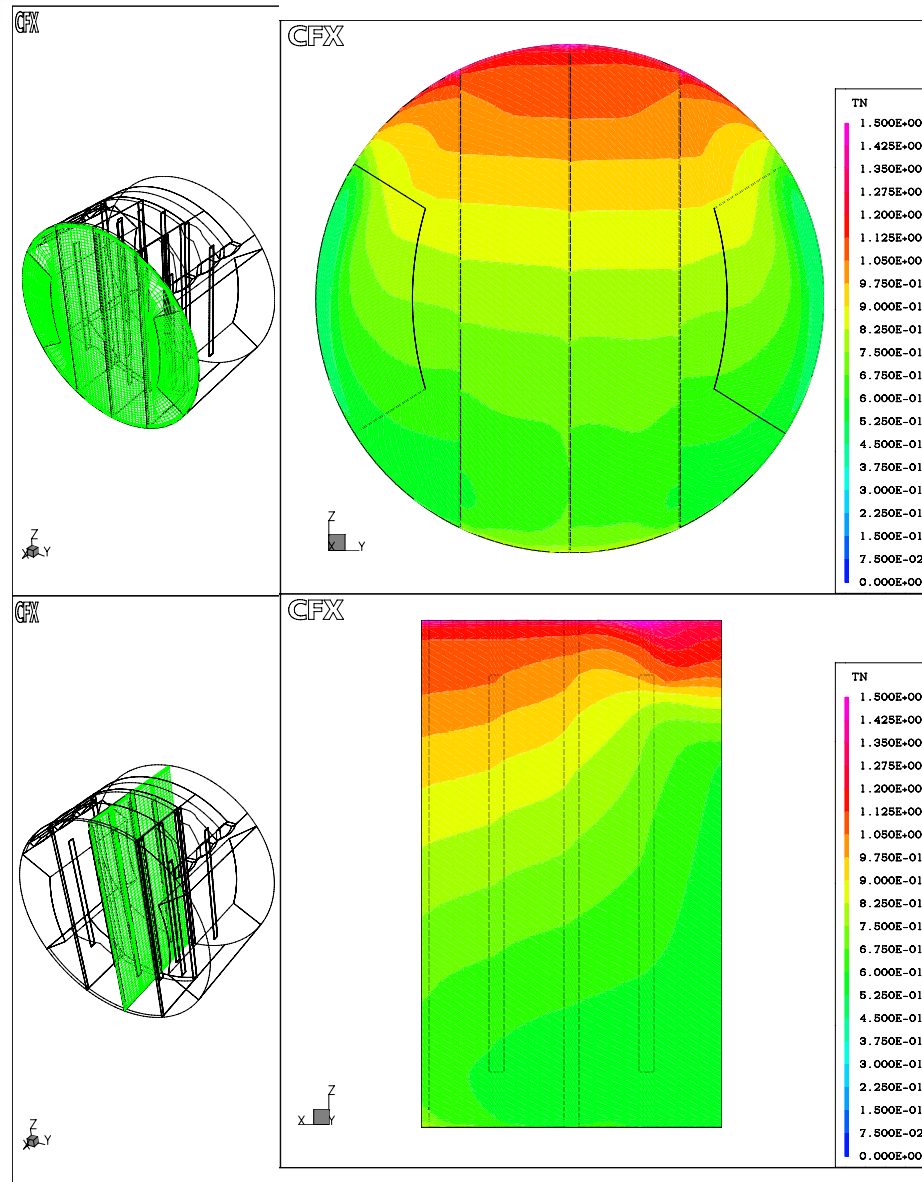
# ACR Moderator Design Assessment (MODTURC\_CLAS Grid)







MODTURC\_CLAS  
predictions of  
moderator temperature  
distributions in ACR  
cross-sectional and  
longitudinal planes





# Summary

- **MODTURC\_CLAS is a well-validated code for the prediction of moderator thermalhydraulics in current CANDU reactors.**
- **A test program in a scaled Moderator Test Facility will extend the MODTURC\_CLAS validation to include the ACR.**



 **AECL**  
TECHNOLOGIES INC.